budget, basin hydrology, Protocol Evaluation Panel (PEP) recommendations, environmental compliance, and other administrative and resource issues pertaining to the AMP.

DATES AND LOCATION: The Glen Canyon Dam Technical Work Group will conduct the following public meeting.

Phoenix, Arizona—September 6–7, 2001. The meeting will begin at 9:30 a.m. and conclude at 5 p.m. on the first day and will begin at 8 a.m. and conclude at 12 noon on the second day. The meeting will be held at the Bureau of Indian Affairs, Western Regional Office, 2 Arizona Center, Conference Rooms A and B (12th floor), 400 North 5th Street, Phoenix, Arizona.

Agenda: The purpose of the meeting will be to discuss the following: FY 2003 work plans, the Automated Generation Control (AGC) Report, results from the Information Needs Workshop, experimental flows, basin hydrology, LIDAR plans, Protocol Evaluation panel (PEP) recommendations, environmental compliance, appraisal of California power situation, nomination of new TWG Chairperson, and other administrative and resource issues pertaining to the AMP.

Agenda items may be revised prior to any of the meetings. Final agendas will be posted 15 days in advance of each meeting and can be found on the Bureau of Reclamation website under Environmental Programs at: http://www.uc.usbr.gov. Time will be allowed on each agenda for any individual or organization wishing to make formal oral comments (limited to 10 minutes) at the meetings.

ADDRESSES: To allow full consideration of information by the AMWG and TWG members, written notice must be provided to Randall Peterson, Bureau of Reclamation, Upper Colorado Regional Office, 125 South State Street, Room 6107, Salt Lake City, Utah 84138; E-mail at rpeterson@uc.usbr.gov at least FIVE (5) days prior to the meeting. Any written comments received will be provided to the AMWG and TWG members at the meetings.

FOR FURTHER INFORMATION CONTACT:

Randall Peterson, telephone (801) 524–3758; faxogram (801) 524–3858; rpeterson@uc.usbr.gov.

Dated: July 31, 2001.

Rick L. Gold,

Regional Director.

[FR Doc. 01–20876 Filed 8–17–01; 8:45 am]

BILLING CODE 4310-MN-M

INTERNATIONAL TRADE COMMISSION

[USITC SE-01-030]

Sunshine Act Meeting

AGENCY HOLDING THE MEETING:

International Trade Commission.

TIME AND DATE: August 22, 2001 at 11:00 a.m.

PLACE: Room 101, 500 E Street SW., Washington, DC 20436, Telephone: (202) 205–2000.

STATUS: Open to the public.

MATTERS TO BE CONSIDERED:

- 1. Agenda for future meeting: none.
- 2. Minutes.
- 3. Ratification List.
- 4. Inv. No. 731–TA–894 (Final) (Certain Ammonium Nitrate from Ukraine)—briefing and vote. (The Commission is currently scheduled to transmit its determination and Commissioners' opinions to the Secretary of Commerce on August 31, 2001.)
- 5. Inv. No. NAFTA-312-1 (Certain Steel Wire Rod)—briefing and vote. (The Commission is currently scheduled to transmit its findings to the President on August 23, 2001; Commissioners' opinions are currently scheduled to be transmitted to the President on September 7, 2001.)
- 6. Outstanding action jackets: none. In accordance with Commission policy, subject matter listed above, not disposed of at the scheduled meeting, may be carried over to the agenda of the following meeting.

By order of the Commission. Issued: August 14, 2001.

Donna R. Koehnke,

Secretary.

[FR Doc. 01–21090 Filed 8–16–01; 3:33 pm] $\tt BILLING\ CODE\ 7020–02-M$

DEPARTMENT OF LABOR

Office of the Secretary

Senior Executive Service; Appointment of Members to the Performance Review Board

Title 5 U.S.C. 4314(c)(4) provides that Notice of the appointment of an individual to serve as a member of the Performance Review Board of the Senior Executive Service shall be published in the **Federal Register**.

The following individuals are hereby appointed to a three-year term on the Department's Performance Review Board: D. Cameron Findlay, Chairperson, James Benages, David Dye,

Robert Elam, Richard Fiore, Joe Kennedy, Patrick Pizzella, Paula White.

FOR FURTHER INFORMATION CONTACT: $\ensuremath{Ms}.$

Tali R. Stepp, Director of Human Resources, Room C–5526, U.S. Department of Labor, Frances Perkins Building, 200 Constitution Avenue, N.W., Washington, D.C. 20210, telephone: (202) 693–7600.

Signed at Washington, D.C., this 13th day of August, 2001.

Elaine L. Chao,

Secretary of Labor.

[FR Doc. 01-20923 Filed 8-17-01; 8:45 am]

BILLING CODE 4510-23-M

NATIONAL COMMISSION ON LIBRARIES AND INFORMATION SCIENCE (NCLIS)

Sunshine Act Meeting

AGENCY: National Commission on Libraries and Information Science.

ACTION: Notice of meeting.

SUMMARY: The U.S. National Commission on Libraries and Information Science is holding an open business meeting to discuss administrative and other matters, including development of a program and strategy related to library and information services for individuals with disabilities.

DATE AND TIME: NCLIS Business Meeting—September 12, 2001, 2:00 p.m. to 5:00 p.m.

ADDRESSES: Meeting location—Board Room, Fuller Conference Center, Old. Sturbridge Village, One Sturbridge Village Road, Sturbridge, Massachusetts 01566.

STATUS: Open meeting.

FOR FURTHER INFORMATION CONTACT:

Rosalie Vlach, Director, Legislative and Public Affairs, U.S. National Commission on Libraries and Information Science, 1110 Vermont Avenue, NW., Suite 820, Washington, DC 20005, e-mail *rvlach@nclis.gov*. fax 202–606–9203 or telephone 202–606– 9200.

SUPPLEMENTARY INFORMATION: The meeting is open to the public, subject to space availability. To make special arrangements for physically challenged persons, contact Rosalie Vlach, Director, Legislative and Public Affairs, 1110 Vermont Avenue, NW., Suite 820, Washington, DC 20005, e-mail rvlach@nclis,gov. fax 202–606–9203 or telephone 202–606–9200.

Dated: August 13, 2001.

Judith C. Russell,

NCLIS Deputy Director.

[FR Doc. 01–21025 Filed 8–16–01; 12:28 pm]

BILLING CODE 7527-\$\$-M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-423]

Dominion Nuclear Connecticut, Inc., Millstone Nuclear Power Station, Unit No. 3; Exemption

1.0 Background

The Dominion Nuclear Connecticut, Inc., (the licensee) is the holder of Facility Operating License No. NPF–49 which authorizes operation of the Millstone Nuclear Power Station, Unit No.3 (MP3). The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

The facility consists of a pressurized water reactor located in New London County, Connecticut.

2.0 Request/Action

Title 10 of the Code of Federal Regulations (10 CFR) Part 50, Appendix G, requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak rate testing conditions. Specifically, 10 CFR Part 50, Appendix G states that "[t]he appropriate requirements on * * * the pressure-temperature limits and minimum permissible temperature must be met for all conditions." Appendix G of 10 CFR Part 50 specifies that the requirements for these limits are the American Society of Mechanical Engineers (ASME) Code, Section XI, Appendix G Limits.

To address provisions of amendments to the technical specifications (TSs) P-T limits in the submittal dated April 23, 2001, the licensee requested that the staff exempt MP3 from application of specific requirements of 10 CFR Part 50, Section 50.60(a) and Appendix G, and substitute use of ASME Code Case N-640. Code Case N-640 permits the use of an alternate reference fracture toughness (K_{lc} fracture toughness curve instead of K_{la}. fracture toughness curve) for reactor vessel materials in determining the P-T limits. Since the K_{lc} fracture toughness curve shown in ASME Section XI, Appendix A, Figure A-2200-1 provides greater allowable fracture toughness than the corresponding Kla fracture toughness

curve of ASME Section XI, Appendix G, Figure G–2210–1, using the $K_{\rm lc}$ fracture toughness, as permitted by Code Case N–640, in establishing the P–T limits would be less conservative than the methodology currently endorsed by 10 CFR Part 50, Appendix G. Considering this, an exemption to apply the Code Case would be required by 10 CFR 50.60.

The licensee proposed to revise the P–T limits in the TSs for MP3 using the $K_{\rm lc}$ fracture toughness curve, in lieu of the $K_{\rm la}$ fracture toughness curve, as the lower bound for fracture toughness.

Use of the K_{lc} curve in determining the lower bound fracture toughness in the development of P-T operating limits curve is more technically correct than the K_{la} curve since the rate of loading during a heatup or cooldown is slow and is more representative of a static condition than a dynamic condition. The K_{lc} curve appropriately implements the use of static initiation fracture toughness behavior to evaluate the controlled heatup and cooldown process of a reactor vessel. The staff has required use of the initial conservatism of the K_{la} curve since 1974 when the curve was codified. This initial conservatism was necessary due to the limited knowledge of RPV materials. Since 1974, additional knowledge has been gained about RPV materials, which demonstrates that the lower bound on fracture toughness provided by the K_{la} curve is well beyond the margin of safety required to protect the public health and safety from potential RPV failure. In addition, P-T curves based on the K_{lc} curve will enhance overall plant safety by opening the P-T operating window with the greatest safety benefit in the region of low temperature operations.

In summary, the ASME Section XI, Appendix G, procedure was conservatively developed based on the level of knowledge existing in 1974 concerning RPV materials and the estimated effects of operation. Since 1974, the level of knowledge about these topics has been greatly expanded. The Commission concurs that this increased knowledge permits relaxation of the ASME Section XI, Appendix G requirements by applying the K_{lc} fracture toughness, as permitted by Code Case N-640, while maintaining, pursuant to 10 CFR 50.12(a)(2)(ii), the underlying purpose of the ASME Code and the NRC regulations to ensure an acceptable margin of safety.

3.0 Discussion

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR Part 50, when (1) the exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) when special circumstances are present. The staff considers that pursuant to 10 CFR 50.12(a)(2)(ii) special circumstances are present and that an exemption may be granted to allow use of the methodology of Code Case N-640 to revise the P-T limits for MP3 because it would provide an adequate margin of safety against brittle fracture. See the safety evaluation supporting these findings dated August 14, 2001.

4.0 Conclusion

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemption is authorized by law, will not endanger life or property or common defense and security, and is, otherwise, in the public interest. Also, special circumstances are present. Therefore, the Commission hereby grants Dominion Nuclear Connecticut, Inc., an exemption from the requirements of 10 CFR 50.60(a) and 10 CFR Part 50, Appendix G, for MP3. Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (66 FR 42567).

This exemption is effective upon issuance

Dated at Rockville, Maryland, this 14th day of August.

For the Nuclear Regulatory Commission. **John A. Zwolinski**,

Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 01–20886 Filed 8–17–01; 8:45 am] **BILLING CODE 7590–01–P**

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-445 AND 50-446]

TXU Electric, Comanche Peak Steam Electric Station, Units 1 and 2; Notice of Consideration of Approval of Transfer of Facility Operating Licenses and Conforming Amendments, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (NRC or the Commission) is considering the issuance of an order under 10 CFR 50.80 approving the transfer of Facility Operating License Nos. NPF–87 and NPF–89 for Comanche Peak Steam Electric Station (CPSES),