

For The Nuclear Regulatory Commission.

Dated at Rockville, Maryland, this 27th day of October, 1999.

E. William Brach,

Director, Spent Fuel Project Office, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 99-29125 Filed 11-5-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-423-LA-3; ASLBP No. 00-771-01-LA]

In the Matter of Northeast Nuclear Energy Company (Millstone Nuclear Power Station, Unit No. 3; Facility Operating License NPF-49); Notice of Prehearing Conference

November 2, 1999.

This proceeding involves the proposed increase in capacity (through the addition of high-density storage racks) of the spent fuel storage pool of the Millstone Nuclear Power Station, Unit No. 3, in New London County, Connecticut. Notice is hereby given that, as described in the Atomic Safety and Licensing Board's Memorandum and Order (Intervention Petition), dated October 28, 1999, a prehearing conference is hereby scheduled for Monday, December 13, 1999, beginning at 2 p.m., and continuing (to the extent necessary) on Tuesday, December 14, 1999, at the Ernst Common Room, Connecticut College, 270 Mohegan Avenue, New London, Connecticut 06320.

The conference will consider the standing of and proposed contentions submitted jointly by the Connecticut Coalition Against Millstone (CCAM) and the Long Island Coalition Against Millstone (CAM), petitioners for intervention in this proceeding. To the extent appropriate, the conference will also consider proposed discovery and hearing schedules, possibilities of settlement of various issues, and other procedural matters as may aid in the orderly disposition of the proceeding.

During the course of this proceeding, the Licensing Board, pursuant to 10 CFR 2.715(a), will entertain written or oral limited appearance statements, from any person who is not a party to the proceeding or a petitioner for intervention. The Licensing Board does not plan to entertain such statements at the December 13-14, 1999 prehearing conference. Written statements, and requests to make oral statements, should be submitted to the Office of the Secretary, Rulemaking and Adjudications Staff, U.S. Nuclear Regulatory Commission, Washington,

DC 20555. A copy of such statement or request should also be served on the Chairman of this Atomic Safety and Licensing Board, T3 F23, U.S. Nuclear Regulatory Commission, Washington, DC 20555, or CXB2@nrc.gov.

Documents related to this proceeding, issued prior to December 1, 1999, are available in print form for public inspection at the Commission's Public Document Room (PDR), 2120 L St. NW, Washington, DC. Documents issued prior to November 1, 1999 may be found in microfiche format at the Learning Resources Center, Three Rivers Community-Technical College, 574 New London Turnpike, Norwich, Connecticut (860-885-2346); at the Russell Library, 123 Broad Street, Middletown, Connecticut (860-347-2520); and, in late November, at the Connecticut State Library, Federal Documents, 231 Capitol Avenue, Hartford, Connecticut (860-566-4889). Documents issued subsequent to November 1, 1999, are available electronically through the Agencywide Documents Access and Management System (ADAMS), with access to the public through NRC's Internet Web site (Public Electronic Reading Room Link, <<http://www.nrc.gov/NRC/ADAMS/index.html>>). The PDR and the majority of public libraries have terminals for public access to the Internet.

Rockville, Maryland, November 2, 1999.

For the Atomic Safety and Licensing Board.

Charles Bechhoefer,

Chairman, Administrative Judge.

[FR Doc. 99-29120 Filed 11-5-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-333]

Power Authority of the State of New York; Notice of Consideration of Issuance of Amendment To Facility Operating License and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-59, issued to the Power Authority of the State of New York, (PASNY or the licensee), for operation of the James A. FitzPatrick Nuclear Power Plant, (FitzPatrick), located in Oswego County, New York.

The proposed amendment, requested by the licensee in a letter dated March 31, 1999, as supplemented by letters dated May 20, June 1, July 14, and

October 14, 1999, represent a full conversion from the current Technical Specifications (CTS) to a set of Improved Technical Specifications (ITS) based on NUREG-1433, "Standard Technical Specifications (STS) for General Electric Plants, BWR/4" Revision 1, dated April 1995. NUREG-1433 has been developed by the Commission's staff through working groups composed of both NRC staff members and industry representatives, and has been endorsed by the staff as part of an industry-wide initiative to standardize and improve the Technical Specifications (TS) for nuclear power plants. As part of this submittal, the licensee has applied the criteria contained in the Commission's "Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors (Final Policy Statement)," published in the **Federal Register** on July 22, 1993 (58 FR 39132), to the CTS, and, using NUREG-1433 as a basis, proposed an ITS for FitzPatrick. The criteria in the Final Policy Statement were subsequently added to 10 CFR 50.36, "Technical Specifications," in a rule change that was published in the **Federal Register** on July 19, 1995 (60 FR 36953) and became effective on August 18, 1995.

The licensee has categorized the proposed changes to the CTS into four general groupings. These groupings are characterized as administrative changes, relocated changes, more restrictive changes and less restrictive changes.

Administrative changes are those that involve restructuring, renumbering, rewording, interpretation and complex rearranging of requirements and other changes not affecting technical content or substantially revising an operating requirement. The reformatting, renumbering and rewording process reflects the attributes of NUREG-1433 and does not involve technical changes to the CTS. The proposed changes include: (a) Providing the appropriate numbers, *etc.*, for NUREG-1433 bracketed information (information that must be supplied on a plant-specific basis, and which may change from plant to plant), (b) identifying plant-specific wording for system names, *etc.*, and (c) changing NUREG-1433 section wording to conform to existing licensee practices. Such changes are administrative in nature and do not impact initiators of analyzed events or assumed mitigation of accident or transient events.

Relocated changes are those involving relocation of requirements and surveillances for structures, systems, components, or variables that do not meet the criteria for inclusion in TS.

Relocated changes are those CTS requirements that do not satisfy or fall within any of the four criteria specified in the 10 CFR 50.36(c)(2)(ii) and may be relocated to appropriate licensee-controlled documents.

The licensee's application of the screening criteria is described in the attachment of the licensee's March 31, 1999, submittal, which is entitled, "Application of NRC Selection Criteria to James A. FitzPatrick Nuclear Power Plant Technical Specifications" (Split Report) in Volume 1 of the submittal. The affected structures, systems, components or variables are not assumed to be initiators of analyzed events and are not assumed to mitigate accident or transient events. The requirements and surveillances for these affected structures, systems, components, or variables will be relocated from the TS to administratively controlled documents such as the quality assurance program, the final safety analysis report (FSAR), the ITS BASES, the Technical Requirements Manual (TRM) that is incorporated by reference in the FSAR, the Core Operating Limits Report (COLR), the Offsite Dose Calculation Manual (ODCM), the Inservice Testing (IST) Program, or other licensee-controlled documents. Changes made to these documents will be made pursuant to 10 CFR 50.59 or other appropriate control mechanisms, and may be made without prior NRC review and approval. In addition the affected structures, systems, components, or variables are addressed in existing surveillance procedures that are also subject to 10 CFR 50.59. These proposed changes will not impose or eliminate any requirements.

More restrictive changes are those involving more stringent requirements compared to the CTS for operation of the facility. These more stringent requirements do not result in operation that will alter assumptions relative to the mitigation of an accident or transient event. The more restrictive requirements will not alter the operation of process variables, structures, systems, and components described in the safety analyses. For each requirement in the STS that is more restrictive than the CTS that the licensee proposes to adopt in the ITS, the licensee has provided an explanation as to why it has concluded that adopting the more restrictive requirement is desirable to ensure safe operation of the facility because of specific design features of the plant.

Less restrictive changes are those where CTS requirements are relaxed or eliminated, or new plant operational flexibility is provided. The more

significant "less restrictive" requirements are justified on a case-by-case basis. When requirements have been shown to provide little or no safety benefit, their removal from the TS may be appropriate. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of (a) generic NRC actions, (b) new NRC staff positions that have evolved from technological advancements and operating experience, or (c) resolution of the Owners Groups' comments on the Improved Standard Technical Specifications. Generic relaxations contained in NUREG-1433 were reviewed by the staff and found to be acceptable because they are consistent with current licensing practices and NRC regulations. The licensee's design is being reviewed to determine if the specific design basis and licensing basis are consistent with the technical basis for the model requirements in NUREG-1433, thus providing a basis for the ITS, or if relaxation of the requirements in the CTS is warranted based on the justification provided by the licensee.

These administrative, relocated, more restrictive, and less restrictive changes to the requirements of the CTS do not result in operations that will alter assumptions relative to mitigation of an analyzed accident or transient event.

In addition to the proposed changes solely involving the conversion, there are also changes proposed that are differences to the requirements in both the CTS and the Standard Technical Specifications (STS) NUREG-1433. These proposed beyond-scope issues to the ITS conversion are as follows:

1. ITS 3.0.3, Limiting Condition for Operation (LCO) to be in MODE 2 was changed to allow a 9-hour completion time.

2. ITS 3.3.1.1, Reactor Protection System (RPS) Instrumentation Function 5, reactor scram on main steam isolation valve (MSIV) closure. The trip setting valve was changed from less than or equal to 10 percent (in the CTS) to less than or equal to 14 percent in the ITS.

3. ITS 3.3.1.1, Extending Required Action F.1 Completion Time from 6 hours to 8 hours for consistency with Current Licensing Basis (CLB) and changing 3.0.3 which allows 8 hours to be in MODE 2 after initiation of Action.

4. ITS 3.3.5.1, Automatic Depressurization System (ADS) initiation timer and the Containment Spray (CS) and Low-Pressure Coolant Injection (LPCI) pump start timer values were changed from the CTS and the STS and tolerances relaxed to allow the extension of CALIBRATION Frequency to 24 months in the ITS.

5. ITS 3.3.5.1, CS, LPCI and ADS Logic System Functional Test (LSFT) Frequency was extended from 18 months (in the CTS) to 24 months in the ITS.

6. ITS 3.4.9, Reactor Coolant System (RCS) Pressure/Temperature (P/T) Limits in CTS were changed to add a new alternate criteria in ITS to allow idle recirculating pump (loop) start if the operating loop is greater than 40 percent flow or if the idle loop is less than 40% flow for less than or equal to 30 minutes.

7. ITS 3.5.1, ECCS-Operating, High-Pressure Coolant Injection (HPCI) and LPCI pump flow rates in CTS were reduced to SAFER/GESTR-Loss-of-Coolant Accident (LOCA) flow rates in the ITS.

8. ITS 3.5.2, ECCS-Shutdown, reduced Residual Heat Removal (RHR) LPCI pump flow rates in CTS to SAFER/GESTR-LOCA flow rates as in ITS 3.5.1 for RHR LPCI pumps.

9. ITS 3.8.1, AC Sources—Operating, Condition D for two reserve circuits inoperable in CTS was changed to add new interim power reduction to less than or equal to 45 percent with a 36-hour Completion Time in the ITS.

10. ITS 3.8.4, DC Sources—Operating (in CTS) was changed to allow 8 hours to restore one inoperable source in the ITS.

11. ITS 5.5, changed Standby Gas Treatment (SGT) and Control Room Emergency Ventilation Air Supply (CREVAS) system filter testing (in the CTS) from 6 months (or 12 months) to 24 months in the ITS for consistency with Regulatory Guide 1.52, Revision 2 or the fuel cycle length.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

By December 8, 1999, the licensee may file a request for a hearing with respect to issuance of the amendment to the NMP2 operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and accessible electronically through

the ADAMS Public Electronic Reading Room link at the NRC Web site (<http://www.NRC.gov>). If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law

or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Mr. David E. Blabey, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CAR 2.714(a)(1)(I)-(v) and 2.714(d).

If a request for a hearing is received, the Commission's staff may issue the amendment after it completes its technical review and prior to the completion of any required hearing if it publishes a further notice for public comment of its proposed finding of no significant hazards consideration in accordance with 10 CFR 50.91 and 50.92.

For further details with respect to this action, see the application for amendment dated March 31, 1999, as supplemented by letters dated May 20, June 1, July 14, and October 14, 1999, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and accessible electronically through the ADAMS Public Electronic Reading Room link at the NRC Web site (<http://www.NRC.gov>).

Dated at Rockville, MD, this 3rd day of November 1999.

For the Nuclear Regulatory Commission.

Guy S. Vissing,

Sr. Project Manager, Section 1, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-29121 Filed 11-5-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72-2]

Virginia Electric and Power Co., Notice of Docketing of the Materials License SNM-2501 Amendment Application for the Surry Independent Spent Fuel Storage Installation

By letter dated April 5, 1999, Virginia Electric and Power Company (Virginia Power) submitted an application to the Nuclear Regulatory Commission (NRC or the Commission) in accordance with 10 CFR part 72 requesting the amendment of the Surry Power Station independent spent fuel storage installation (ISFSI) license (SNM-2501) and the Technical Specifications for the ISFSI located in Surry County, Virginia. Virginia Power is seeking Commission approval to amend the materials license and ISFSI Technical Specifications to permit the storage of burnable poison rod assemblies and thimble plug devices in the GNSI CASTOR V/21, Westinghouse MC-10, and NAC-I28 storage casks at the Surry ISFSI.

This application was docketed under 10 CFR part 72; the ISFSI Docket No. is 72-2 and will remain the same for this action. The amendment of an ISFSI license is subject to the Commission's approval.

The Commission may issue either a notice of hearing or a notice of proposed action and opportunity for hearing in accordance with 10 CFR 72.46(b)(1) or, if a determination is made that the amendment does not present a genuine issue as to whether public health and safety will be significantly affected, take immediate action on the amendment in accordance with 10 CFR 72.46(b)(2) and provide notice of the action taken and an opportunity for interested persons to request a hearing on whether the action should be rescinded or modified.

For further details with respect to this application, see the application dated April 5, 1999, which is available for public inspection at the Commission's Public Document Room, 2120 L Street, NW, Washington, DC 20555 and at the local Public Document Room located at