Brief description of amendment: The amendment proposes to revise Technical Safety Requirement (TSR) 2.3.4.7, Criticality Accident Alarm System (CAAS), Required Action A.1.5 to provide additional time to operate the withdrawal station in normal steady state operation should the alarm system be declared inoperable. This would allow the accumulators in the product withdrawal area to be filled while the CAAS was inoperable instead of immediately placing the cascade into the recycle mode.

Basis for finding of no significance: 1. The proposed amendment will not result in a change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed changes to the TSR to provide additional time to conduct operations when the CAAS is inoperable will have no effect on the generation or disposition of effluents. Therefore, the proposed TSR modification will not result in a change to the types or amount of effluents that may be released offsite.

2. The proposed amendment will not result in a significant increase in individual or cumulative occupational radiation exposure.

The CAAS does not prevent criticality, therefore, the possibility of a criticality occurring during the period of CAAS inoperability is not increased. Personnel access during the period of inoperability is limited and individuals are required to have an alternate means of criticality alarm notification. However, in the unlikely event a criticality did occur during this period, the personnel notification might not be as prompt as the CAAS. Therefore, the potential radiation exposure for an individual could be higher because the individual remained in the area for a longer period of time. This slight chance for increased exposure is not considered to be significant. The proposed changes will not significantly increase any exposure to radiation due to normal operations. Therefore, the changes will not result in a significant increase in individual or cumulative radiation

3. The proposed amendment will not result in a significant construction impact.

The proposed changes will not result in any construction, therefore, there will be no construction impacts.

4. The proposed amendment will not result in a significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents.

The proposed change to TSR 2.3.4.7 to allow the accumulators to be filed in the event of CAAS inoperability does not increase the probability of any accident. It is possible that personnel exposure could be slightly increased due to possible short delays in personnel notification. For personnel in the immediate vicinity of any criticality, the consequences would not be expected to change. Consequences to the facility would not be changed. These changes will not significantly increase the probability of occurrence or consequence of any postulated accident currently identified in the safety analysis report.

5. The proposed amendment will not result in the possibility of a new or different kind of accident.

The proposed TSR modification will allow the routine operation of filling an accumulator to occur while the CAAS is inoperable. This change does not introduce any new or different accidents than those previously analyzed. Therefore, the proposed changes will not create the possibility of a different type of equipment malfunction or a different type of accident.

6. The proposed amendment will not result in a significant reduction in any

margin of safety.

The proposed changes do not change the types of accidents that could occur or the probability of any accidents. The margin of safety for withdrawal related operations is not changed. Criticality detection would be provided through the use of personnel alarming devices. The changes do not significantly decrease the margins of safety.

7. The proposed amendment will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs.

Implementation of the proposed changes do not change the safety, safeguards, or security programs. Therefore, the effectiveness of the safety, safeguards, and security programs is not decreased.

Effective date: The amendment to Certificate of Compliance GDP-1 becomes effective 15 days after being signed by the Director, Office of Nuclear Material Safety and Safeguards.

Certificate of Compliance No. GDP-1: Amendment will revise TSR 2.3.4.7 to provide additional time to operate the withdrawal station in normal steady state operation should the CAAS be declared inoperable.

Local Public Document Room location: Paducah Public Library, 555 Washington Street, Paducah, Kentucky 42003.

Dated at Rockville, Maryland, this 7th day of 1998.

For the Nuclear Regulatory Commission. **Carl J. Paperiello**,

Director, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98–33208 Filed 12–14–98; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Consolidated Guidance About Materials Licenses: Program-Specific Guidance About Self-Shielded Irradiator Licenses, Dated October 1998

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Notice of availability.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is announcing the availability of NUREG-1556, Volume 5, "Consolidated Guidance about Materials Licenses: Program-Specific Guidance about Self-Shielded Irradiator Licenses," dated October 1998.

ADDRESSES: Copies of NUREG-1556, Vol. 5, may be obtained by writing to the Superintendent of Documents, U.S.

Vol. 5, may be obtained by writing to the Superintendent of Documents, U.S. Government Printing Office, P.O. Box 37082, Washington, DC 20402–9328. Copies are also available from the National Technical Information Service, 5285 Port Royal Road, Springfield, Virginia 22161. A copy of the document is also available for inspection and/or copying for a fee in the NRC Public Document Room, 2120 L Street, NW. (Lower Level), Washington, DC 20555–0001.

FOR FURTHER INFORMATION CONTACT: Ms. Sally L. Merchant, Mail Stop TWFN 9–F–31, Division of Industrial and Medical Nuclear Safety, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone: (301) 415–7874.

SUPPLEMENTARY INFORMATION: On December 23, 1997 (62 FR 67100), NRC announced the availability of draft NUREG-1556, Volume 5, "Consolidated Guidance about Materials Licenses: Program-Specific Guidance about Self-Shielded Irradiator Licenses," dated October 1997, and requested comments on it. This draft NUREG report was the fifth program-specific guidance developed to support an improved materials licensing process. The NRC staff considered all the comments, including constructive suggestions to improve the document, in the preparation of the final NUREG report.

The final version of NUREG-1556, Volume 5, is now available for use by applicants, licensees, NRC license

reviewers, and other NRC staff. It supersedes the guidance for applicants and licensees previously found in Regulatory Guide 10.9, "Guide for the Preparation of Applications for Licenses for the Use of Self-Contained Dry Source-Storage Gamma Irradiators," dated December 1988, and the guidance for licensing staff previously found in Policy and Guidance Directive, FC 84-16, Revision 1, "Standard Review Plan for Applications for Use of Self-Contained Dry Source-Storage Gamma Irradiators," dated January 26, 1989. In addition, this draft report also contains information found in pertinent Technical Assistance Requests and Information Notices. NRC staff will use this final report in reviewing these applications.

Electronic Access

NUREG-1556, Volume 5, will also be available electronically approximately 1 month after publication of this notice by visiting NRC's Home Page (http://www.nrc.gov) and choosing "Nuclear Materials," and then "NUREG-1556, Volume 5."

Small Business Regulatory Enforcement Fairness Act

In accordance with the Small Business Regulatory Enforcement Act of 1996, NRC has determined that this action is not a major rule and has verified this determination with the Office of Information and Regulatory Affairs of the Office of Management and Budget.

Dated at Rockville, Maryland, this 7th day of December, 1998.

For the Nuclear Regulatory Commission. **Donald A. Cool**,

Director, Division of Industrial and Medical Nuclear Safety, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 98–33202 Filed 12–14–98; 8:45 am] BILLING CODE 7590–01–P

PENSION BENEFIT GUARANTY CORPORATION

Interest Assumption for Determining Variable-Rate Premium; Interest Assumptions for Multiemployer Plan Valuations Following Mass Withdrawal

AGENCY: Pension Benefit Guaranty Corporation.

ACTION: Notice of interest rates and assumptions.

rates and assumptions are published elsewhere (or are derivable from rates published elsewhere), but are collected and published in this notice for the convenience of the public. Interest rates are also published on the PBGC's web site (http://www.pbgc.gov).

DATES: The interest rate for determining the variable-rate premium under part 4006 applies to premium payment years beginning in December 1998. The interest assumptions for performing multiemployer plan valuations following mass withdrawal under part 4281 apply to valuation dates occurring in January 1999.

FOR FURTHER INFORMATION CONTACT: Harold J. Ashner, Assistant General Counsel, Office of the General Counsel, Pension Benefit Guaranty Corporation, 1200 K Street, NW., Washington, DC 20005, 202–326–4024. (For TTY/TDD users, call the Federal relay service toll-free at 1–800–877–8339 and ask to be connected to 202–326–4024.)

SUPPLEMENTARY INFORMATION:

Variable-Rate Premiums

Section 4006(a)(3)(E)(iii)(II) of the **Employee Retirement Income Security** Act of 1974 (ERISA) and § 4006.4(b)(1) of the PBGC's regulation on Premium Rates (29 CFR part 4006) prescribe use of an assumed interest rate in determining a single-employer plan's variable-rate premium. The rate is the "applicable percentage" (currently 85 percent) of the annual yield on 30-year Treasury securities for the month preceding the beginning of the plan year for which premiums are being paid (the 'premium payment year''). The yield figure is reported in Federal Reserve Statistical Releases G.13 and H.15.

The assumed interest rate to be used in determining variable-rate premiums for premium payment years beginning in December 1998 is 4.46 percent (*i.e.*, 85 percent of the 5.25 percent yield figure for November 1998).

The following table lists the assumed interest rates to be used in determining variable-rate premiums for premium payment years beginning between January and December 1998.

For premium payment years beginning in:	The as- sumed in- terest rate is:
January 1998	5.09
February 1998	4.94
March 1998	5.01
April 1998	5.06
May 1998	5.03
June 1998	5.04
July 1998	4.85
August 1009	1 02

For premium payment years be-	The as- sumed in-
ginning in:	terest rate is:
September 1998	4.71
October 1998	4.42
November 1998	4.26
December 1998	4.46

Multiemployer Plan Valuations Following Mass Withdrawal

The PBGC's regulation on Duties of Plan Sponsor Following Mass Withdrawal (29 CFR part 4281) prescribes the use of interest assumptions under the PBGC's regulation on Allocation of Assets in Single-employer Plans (29 CFR part 4044). The interest assumptions applicable to valuation dates in January 1999 under part 4044 are contained in an amendment to part 4044 published elsewhere in today's **Federal Register**. Tables showing the assumptions applicable to prior periods are codified in appendix B to 29 CFR part 4044.

Issued in Washington, DC, on this 9th day of December 1998.

John Seal.

Acting Executive Director, Pension Benefit Guaranty Corporation.

[FR Doc. 98–33138 Filed 12–14–98; 8:45 am]

POSTAL RATE COMMISSION

Sunshine Act Meeting

NAME OF AGENCY: Postal Rate Commission.

TIME AND DATE: 3:00 p.m., December 10, 1998.

PLACE: Commission Conference Room, 1333 H Street, NW, Suite 300, Washington, DC 20268–0001.

STATUS: Closed.

MATTERS TO BE CONSIDERED: Personnel Issues.

CONTACT PERSON FOR MORE INFORMATION:

Stephen L. Sharfman, General Counsel, Postal Rate Commission, Suite 300, 1333 H Street, NW, Washington, DC 20268–0001, (202) 789–6840.

Dated: December 11, 1998.

Margaret P. Crenshaw,

Secretary.

[FR Doc. 98–33253 Filed 12–11–98; 10:44 aml

BILLING CODE 7710-FW-M