be provided in the desired format, reporting burden (time and financial resources) is minimized, collection instruments are clearly understood, and impact of collection requirements on respondents can be properly assessed. Currently, the Occupational Safety and Health Administration (OSHA) is soliciting comments concerning the proposed extension of the information collection requirements contained in the standard on Accident Prevention Tags (29 CFR 1910.145). The Agency is particularly interested in comments that:

- Evaluate whether the proposed collection of information is necessary for the proper performance of the functions of the Agency, including whether the information will have practical utility;
- Evaluate the accuracy of the Agency's estimate of the burden of the proposed collection of information, including the validity of the methodology and assumptions used;
- Enhance the quality, utility and clarity of the information to be collected; and
- Minimize the burden of the collection of information on those who are to respond, including through the use of appropriate automated, electronic, mechanical, or other technological collection techniques or other forms of information technology, e.g., permitting electronic submissions of responses.

DATES: Written comments must be submitted on or before September 28, 1998.

ADDRESSES: Comments are to be submitted to the Docket Office, Docket No. ICR–98–33, Occupational Safety and Health Administration, U.S. Department of Labor, Room N–2625, 200 Constitution Avenue, NW., Washington, DC 20210. Telephone: (202) 219–7894. Written comments limited to 10 pages or less in length may also be transmitted by facsimile to (202) 219–5046.

FOR FURTHER INFORMATION CONTACT: Theda Kenney, Directorate of Safety Standards Programs, Occupational Safety and Health Administration, U.S. Department of Labor, Room N–3605, 200 Constitution Avenue, NW., Washington, DC 20210, telephone: (202) 219–8061. A copy of the referenced information collection request is available for inspection and copying in the Docket Office and will be mailed to persons who request copies by telephoning Theda Kenney at (202) 219–8061, extension 100, or Barbara Bielaski at (202) 219–8076, extension 142. For electronic copies of the Information Collection Request on Accident Prevention Tags, contact OSHA's WebPage on the Internet at http://www.osha.gov and click on ''Regulations and Compliance.''

SUPPLEMENTARY INFORMATION:

I. Background

The Occupational Safety and Health Act of 1970 (the Act) authorizes the promulgation of such health and safety standards as are necessary or appropriate to provide safe or healthful employment and places of employment. The statute specifically authorizes information collection by employers as necessary or appropriate for the enforcement of the Act or for developing information regarding the causes and prevention of occupational injuries, illnesses, and accidents.

In the standard on Accident Prevention Tags, information concerning the degree of hazard associated with a workplace condition is used by the employer to select the type of accident prevention tag (sign) to be used on a workplace hazard. The tag (sign) selected will identify the workplace hazard and convey the severity of hazard and any accident prevention instruction to the employee.

II. Current Actions

This notice requests public comment on OSHA's burden hour estimates prior to OSHA seeking Office of Management and Budget (OMB) approval of the information collection requirements contained in the standard on Accident Prevention Tags (29 CFR 1910.145).

Type of Review: Extension of a Currently Approved Collection.

Agency: U.S. Department of Labor, Occupational Safety and Health Administration.

Title: Accident Prevention Tags (29 CFR 1910.145).

OMB Number: 1218–0132.

Agency Number: Docket Number ICR–98–33.

Affected Public: Business or other for-profit; Not for profit institutions; Federal Government; State, local or tribal Government.

Number of Respondents: 112,000.

Frequency: On Occasion.

Average Time Per Response: 3 minutes (.05 hr.).

Estimated Total Burden Hours: 5,600.

Total Annualized Capital/Startup Costs: $0.

Comments submitted in response to this notice will be summarized and included in the request for Office of Management and Budget (OMB) approval of the information collection request. The comments will become a matter of public record.

Signed at Washington, DC, this 21st day of July 1998.

Charles N. Jeffress,

Assistant Secretary of Labor.

[FR Doc. 98–20137 Filed 7–27–98; 8:45 am]

BILLING CODE 4510–26–M

NATIONAL CREDIT UNION ADMINISTRATION

Sunshine Act Meeting

TIME AND DATE: 10:00 a.m., Thursday, July 30, 1998.

PLACE: Board Room, 7th Floor, Room 7047, 1775 Duke Street, Alexandria, VA 22314–3428.

STATUS: Open.

MATTERS TO BE CONSIDERED:

1. Requests from Two (2) Federal Credit Unions to Convert to Community Charters.
2. Request from a Federal Credit Union to Convert to a State Chartered, Non-federally Insured Credit Union.
3. Request from a Credit Union to Convert Insurance.
4. NCUA's FY–98 Mid-Session Budget Review.
5. Proposed Amendment: Section 701.21(g), NCUA's Rules and Regulations, Real Estate Lending.
6. Proposed Amendment: Section 701.23(b), NCUA's Rules and Regulations, Purchase of Eligible Obligations.

RECESS: 11:15 a.m.

TIME AND DATE: 11:30 a.m, Thursday, July 30, 1998.

PLACE: Board Room, 7th Floor, Room, 7047, 1775 Duke Street, Alexandria, VA 22314–3428.

STATUS: Closed.

MATTERS TO BE CONSIDERED:

1. Administrative Action under Sections 205 and 206 of the Federal Credit Union Act and Part 708 of NCUA's Rules & Regulations. Closed pursuant to exemption (8).
2. Administrative Action under Section 206 of the Federal Credit Union Act. Closed pursuant to exemptions (4), (7), (8), (9)(A)(ii) and (9)(B).
3. Administrative Action under Section 206 of the FCU Act. Closed pursuant to exemptions (2) and (8).
4. Administrative Action under Part 704 of NCUA's Rules and Regulations. Closed pursuant to exemption (8).
5. Corporate Examiner Review Task Force Recommendations. Closed pursuant to exemption (2).
6. Appeal from a Federal Credit Union of the Regional Director's Denial of a Community Charter. Closed pursuant to exemption (8).

7. Three (3) Administrative Actions under Part 745 of NCUA's Rules and Regulations. Closed pursuant to exemption (6).

8. Seven (7) Personnel Actions. Closed pursuant to exemptions (2) and (6).

FOR FURTHER INFORMATION CONTACT: Becky Baker, Secretary of the Board, Telephone (703) 518–6304.

Becky Baker,

Secretary of the Board.

[FR Doc. 98–20208 Filed 7–23–98; 4:47 pm]

BILLING CODE 7535–01–M

NUCLEAR REGULATORY COMMISSION

Northern States Power Company; Notice of Issuance of Amendment to Facility Operating License No. DPR–22 Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

[Docket No. 50–263]

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR–22 issued to Northern States Power Company (the licensee) for operation of the Monticello Nuclear Generating Plant located in Wright County, Minnesota.

The proposed amendment would revise Section 3.6.C, Coolant Chemistry, and 3/4.17.B, Control Room Emergency Filtration System, of the Technical Specifications (TS), Appendix A of the Operating License for the Monticello Nuclear Generating Plant. The changes were proposed to establish TS requirements consistent with modified analysis inputs used for the evaluation of the radiological consequences of the main steam line break accident. This amendment request was originally noticed in the **Federal Register** on May 6, 1998 (63 FR 25115). On June 19, 1998, supplemented July 1, 1998, the licensee submitted an application that superseded in its entirety the licensee's previous submittal dated April 11, 1997.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

A limit is established in the plant Technical Specifications for steady state radioiodine concentration in the reactor coolant to ensure that in the event of a release of radioactive material to the environment due to a postulated high energy line break up to and including a design basis Main Steam Line Break Accident, radiation doses are maintained well within the regulatory guidelines. The steady state radioiodine concentration in the reactor coolant is an input for analysis of the radiological consequences of an accident due to a Main Steam Line Break outside of containment and postulated high energy line breaks. In addition, requirements are established in the Technical Specifications for control room habitability. During an accident, the control room emergency filtration system provides filtered air to pressurize the Control Room to minimize the activity, and therefore the radiological dose, inside the control room.

A change is proposed for the steady state radioiodine concentration. This value is conservative with respect to the value used in the Main Steam Line Break dose consequences analysis and is consistent with the dose consequences evaluation of a postulated Reactor Water Cleanup (RWCU) line break. Changes are proposed to the limiting conditions for operation and surveillance requirements for the Control Room Emergency Filtration Train iodine removal efficiency. These changes are consistent with the inputs used in the analysis of the radiological consequences of the postulated RWCU line break and the Main Steam Line Break Accident. Changes to testing requirements are more restrictive and in accordance with the applicable regulatory guidance. These proposed requirements maintain operating restrictions for analytical inputs used in the analysis of the Main Steam Line Break Accident. Evaluation of these events has demonstrated that the postulated radiological consequences will also remain within the licensing basis established in the AEC [Atomic Energy Commission] Provisional Operating License Safety Evaluation Report, dated March 18, 1970, thus the proposed changes do not result in an increase in the consequences of previously evaluated accidents.

The analysis of the Main Steam Line Break Accident performed using a reactor coolant radioiodine concentration of 2 [micro]Ci/gm dose equivalent Iodine-131 and a control room ventilation filter efficiency consistent with the proposed Technical Specifications changes demonstrated that radiological consequences of the Main Steam Line Break are not changed significantly. The radiological consequences of the Main Steam Line Break Accident remain within the exposure guidelines of 10 CFR 100 and 10 CFR 50 Appendix A, General Design Criterion 19. The offsite dose consequences remain bounded by the original licensing basis provided in the AEC Provisional Operating License Safety Evaluation Report, dated March 18, 1970. The control room doses calculated for the hot standby Main Steam Line Break Accident using the TID–14844 dose conversion factors remain bounded by the dose consequences of the comparable design basis loss of coolant accident.

The evaluation of the postulated RWCU line break, performed using a reactor coolant radioiodine concentration of 0.25 [micro]Ci/gm dose equivalent Iodine-131 and a control room ventilation filter efficiency consistent with the proposed Technical Specifications changes, demonstrated that the radiological consequences of this event remain within the exposure guidelines of 10 CFR 100 and 10 CFR 50 Appendix A, General Design Criterion 19. The offsite dose consequences remain bounded by the Main Steam Line Break as established in the licensing basis provided in the AEC Provisional Operating License Safety Evaluation Report, dated March 18, 1970.

The proposed Technical Specification changes do not introduce new equipment operating modes, nor do the proposed changes alter existing system relationships. The proposed changes do not introduce new failure modes. The system improvements to reduce bypass leakage during postulated accidents do not have an adverse effect on control room habitability. Therefore, this amendment will not cause a significant increase in the probability of an accident previously evaluated for the Monticello plant.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The proposed Technical Specification changes do not introduce new equipment operating modes, nor do the proposed changes alter existing system relationships. Operator action to mitigate the consequences of the postulated RWCU line break is conservative based on the simple action required by the operator to close the containment isolation valves within 10 minutes. Isolation at 10 minutes is very conservative since a safety related RWCU containment isolation system that was installed during the 1998 refueling outage would effect an automatic isolation within one minute of the RWCU break.

The proposed change to the specification for reactor coolant dose equivalent radioiodine is conservative with respect to the re-evaluation of the Main Steam Line Break Accident for the more conservative hot standby initial condition for the postulated accident. The proposed change to the specification for reactor coolant dose equivalent radioiodine is consistent with the postulated high energy line break of a Reactor