

vessels (RPVs) are set forth in 10 CFR 50, Appendix G, which incorporates, by reference, P-T limits specified in Appendix G of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI. 10 CFR 50, Appendix G, Section IV.A.2.b, requires that the P-T limits identified in 10 CFR 50, Appendix G, as "ASME Appendix G limits" must be at least as conservative as limits obtained by following the methods of analysis and the margins of safety of the ASME Code, Section XI, Appendix G. 10 CFR 50, Appendix G, Section I, states that "If no edition or addenda are specified, the ASME Code edition and addenda and any limitations and modifications thereof, which are specified in 10 CFR 50.55a, are applicable." With respect to P-T limits, 10 CFR 50, Appendix G, does not specify the edition or addenda of the ASME Code; therefore, the editions and addenda of the ASME Code, Section XI, referred to in 10 CFR 50, Appendix G, are those specified in 10 CFR 50.55a, which include addenda through the 1988 Addenda and editions through the 1989 Edition.

The proposed exemption would allow CP&L to use the 1992 Edition of the ASME Code, Section XI, Appendix A, as an alternative to the 1989 Edition of the ASME Code, Section XI, Appendix G, for determination of BSEP1&2 RPV P-T requirements. The licensee provided information in its application for exemption that demonstrates the equivalency of the proposed alternative method for determining RPV P-T limits to that specified in the 1989 Edition of the ASME Code, Section XI, Appendix G.

The licensee's exemption request and the bases therefore are contained in a CP&L letter dated August 15, 1997. The exemption request is associated with a CP&L application for license amendments for BSEP1&2 dated January 7, 1997, as supplemented on July 25, 1997, and September 15, 1997. That application, which was noticed in the **Federal Register** on March 12, 1997 (62 FR 11485), will—

(1) Correct an error involving a transposition of P-T curves between BSEP1&2.

(2) Replace the current BSEP1&2 RPV hydrostatic test P-T curves for 8, 10, and 12 effective full power years (EFPY) with new 14 and 16 EFPY curves.

The Need for the Proposed Action

CP&L has proposed an alternative to the requirements of 10 CFR 50, Appendix G. In accordance with 10 CFR 50.60(b), an exemption must be granted by the Commission before the proposed alternative may be used by the licensee.

The alternative, and thus the exemption, is needed because CP&L identified typographical errors in equations contained in both the 1989 and 1992 Editions of the ASME Code, Section XI, Appendix G. The alternative of using the 1992 Edition of the ASME Code, Section XI, Appendix A in the determination of P-T limits avoids the problem presented by the typographical errors and achieves a level of safety commensurate to that provided by use of the 1989 Edition of the ASME Code, Section XI, Appendix G. Furthermore, the alternative provides a more efficient means for the licensee to determine the P-T limits for the BSEP1&2 RPVs.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed exemption. The exemption would authorize use of an alternative means for determining RPV P-T limits that is equivalent to that provided by 10 CFR 50, Appendix G and provides a commensurate level of safety.

The proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action involves features located entirely within the restricted area, as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the Commission concludes that there is no significant environmental impact associated with this action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no significant environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of

the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the BSEP dated January 1974.

Agencies and Persons Consulted

In accordance with its stated policy, on September 24, 1997, the staff consulted with the North Carolina State official, Mr. J. James, of the North Carolina Department of Environment, Commerce and Natural Resources, Division of Radiation Protection, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon this environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated August 15, 1997, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the University of North Carolina at Wilmington, William Madison Randall Library, 601 College Road, Wilmington, North Carolina 28403-3297.

Dated at Rockville, Maryland, this 30th day of September, 1997.

For the Nuclear Regulatory Commission.

James E. Lyons,

Director, Project Directorate II-1, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

Review of Dose Modeling Methods for Demonstration of Compliance With the Radiological Criteria for License Termination: Public Workshop

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of workshop.

SUMMARY: The NRC will hold a public workshop in Rockville, Maryland, to provide the NRC staff and the public

with an overview of dose modeling methods that may be useful in demonstrating compliance with dose-based radiological criteria for license termination. The objectives of the workshop are to provide information on Federally-sponsored dose models appropriate for decommissioning assessments, and to discuss selection criteria for evaluating and accepting dose models used to demonstrate compliance with the final rule on "Radiological Criteria for License Termination" (62 FR 39058). This information may be useful in developing regulatory guidance for decommissioning. All interested licensees and members of the public are invited to attend this workshop.

DATES: The workshop will be held on November 13-14, 1997, from 1 p.m. on Thursday, November 13 thru 5:00 p.m., and on November 14 from 9:00 a.m. until about 5 p.m.

Interested parties who are unable to attend the workshop are encouraged to provide written comments on the topic of dose model selection criteria by November 10, 1997.

ADDRESSES: The public workshop will be held in the NRC auditorium at Two White Flint North, 11545 Rockville Pike, Rockville, Maryland.

Information on the workshop program can be viewed, and comments may be posted, electronically, on the NRC Technical Conference Forum Website under the topic "Final Rule for License Termination" at <http://techconf.llnl.gov/cgi-bin/topics>. Comments submitted electronically can also be viewed at that website. The information is also available at the NRC's Public Document Room, 2120 L Street, NW (Lower Level), Washington, DC 20555; telephone 202-634-3273; fax 202-634-3343.

Comments may also be mailed to the Chief, Rules and Directives Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

FOR FURTHER INFORMATION: For information or questions on meeting arrangements, contact Jayne McCausland, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone 301-415-6219, fax 301-415-5385, E-mail: JMM2@NRC.GOV. For technical information or questions, contact Chris Daily, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone 301-415-6026; fax: 301-415-5385; E-mail: CXD@NRC.GOV.

SUPPLEMENTARY INFORMATION: This workshop is one of a series of interactions with the Agreement States, licensees, and the public to gather suggestions and ideas for developing a regulatory guide on "Demonstrating Compliance With the Radiological Criteria for License Termination." The NRC staff are considering a range of dose models that may be appropriate for use in site-specific dose assessments for specific pathways. The workshop will begin with a brief introduction by the NRC staff on the need to use dose models for evaluating compliance with decommissioning guidelines, and the need to develop guidance for evaluating and selecting dose models. The workshop will include formal presentations by invited speakers from the DOE national laboratories, and other Federal Agencies that will address a set of questions developed by the NRC staff focusing on the capability, experience, and appropriateness of each dose model presented. The workshop will conclude with a panel discussion on questions to be considered in selecting dose models related to demonstrating compliance with the radiological criteria for license termination. A summary of the workshop proceedings in the form of a NUREG/CP is planned to be available in the Spring of 1998.

Visitor parking around the NRC building is limited; however, the workshop site is located adjacent to the White Flint Station on the Metro Red Line. Seating for the public will be on a first-come, first-served basis.

A transcript of this workshop will be available for inspection, and copying for a fee at the NRC Public Document Room, 2120 L Street, NW (Lower Level), Washington, DC 20555, on or about December 1, 1997. A copy of the NUREG/CP will also be available in the NRC's Public Document Room later in the Spring of 1998.

Dated at Rockville, Maryland this 25th day of September, 1997.

For the Nuclear Regulatory Commission.

Cheryl A. Trottier,

Chief, Radiation Protection and Health Effects Branch, Division of Regulatory Applications, Office of Nuclear Regulatory Research.

[FR Doc. 97-26159 Filed 10-1-97; 8:45 am]

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SECURITIES AND EXCHANGE COMMISSION

Sunshine Act Meeting

Notice is hereby given, pursuant to the provisions of the Government in the Sunshine Act, Pub. L. 94-409, that the Securities and Exchange Commission

will hold the following meeting during the week of October 6, 1997.

A closed meeting will be held on Tuesday, October 7, 1997, at 2:30 p.m.

Commissioners, Counsel to the Commissioners, the Secretary to the Commission, and recording secretaries will attend the closed meeting. Certain staff members who have an interest in the matters may also be present.

The General Counsel of the Commission, or his designee, has certified that, in his opinion, one or more of the exemptions set forth in 5 U.S.C. 552b(c) (4), (8), (9)(A) and (10) and 17 CFR 200.402(a) (4), (8), (9)(i) and (10), permit consideration of the scheduled matters at the closed meeting. Commissioner Johnson, as duty officer, voted to consider the items listed for the closed meeting in a closed session.

The subject matter of the closed meeting scheduled for Tuesday, October 7, 1997, at 2:30 p.m., will be:

Institution and settlement of injunctive actions.

Institution and settlement of administrative proceedings of an enforcement nature.

At times, changes in Commission priorities require alternations in the scheduling of meeting items. For further information and to ascertain what, if any, matters have been added, deleted or postponed, please contact: the Office of the Secretary at (202) 942-7070.

Dated: September 30, 1997.

Jonathan G. Katz,

Secretary.

[FR Doc. 97-26358 Filed 9-30-97; 4:00 pm]

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SECURITIES AND EXCHANGE COMMISSION

[Release No. 34-39137; File No. SR-OPRA-97-4]

Options Price Reporting Authority; Notice of Filing of Amendment to OPRA Plan Revising OPRA's Dial-Up Market Data Service Rider to its Vendor Agreement to Accommodate the Vendor's Provision of Dial-Up Service to Customers of OPRA Subscribers

September 26, 1997.

Pursuant to Rule 11Aa3-2 under the Securities Exchange Act of 1934 ("Exchange Act"), notice is hereby given that on September 11, 1997, the Options Price Reporting Authority ("OPRA")¹

¹ OPRA is a National Market System Plan approved by the Commission pursuant to Section 11A of the Exchange Act and Rule 11Aa3-2 thereunder. See Securities Exchange Act Release No. 17638 (Mar. 18, 1981).