requirements of 10 CFR Part 50, Appendix R, Section III.O.

IV

Therefore, contingent on the use of the compensatory measures that are itemized in the licensee's December 23, 1996, exemption request, the NRC staff has concluded that the licensee's proposed use of the remote oil addition system will not present an undue risk to public health and safety and is consistent with the common defense and security. The NRC staff has determined that there are special circumstances present, as specified in 10 CFR 50.12(a)(2), in that application of 10 CFR Part 50, Appendix R, Section III.O is not necessary in order to achieve the underlying purpose of this regulation.

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), an exemption is authorized by law, will not endanger life or property or common defense and security, and is, otherwise, in the public interest. Therefore, the Commission hereby grants an exemption from the requirements of 10 CFR Part 50, Appendix R, Section III.O to the extent that the RCP lube oil fill lines are required to be protected with a collection system.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (62 FR 19632).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 14th day of June 1997.

For the Nuclear Regulatory Commission.

Samuel J. Collins,

Director, Office of Nuclear Reactor Regulation.

[FR Doc. 97–16382 Filed 6–20–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket 70–7001]

Notice of Amendment to Certificate of Compliance GDP–1 for the U.S. Enrichment Corporation Paducah Gaseous Diffusion Plant; Paducah, KY

The Director, Office of Nuclear Material Safety and Safeguards, has made a determination that the following amendment request is not significant in accordance with 10 CFR 76.45. In making that determination the staff concluded that: (1) There is no change in the types or significant increase in the amounts of any effluents that may be released offsite; (2) there is no significant increase in individual or cumulative occupational radiation exposure; (3) there is no significant construction impact; (4) there is no significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents; (5) the proposed changes do not result in the possibility of a new or different kind of accident; (6) there is no significant reduction in any margin of safety; and (7) the proposed changes will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs. The basis for this determination for the amendment request is shown below.

The NRC staff has reviewed the certificate amendment application and concluded that it provides reasonable assurance of adequate safety, safeguards, and security, and compliance with NRC requirements. Therefore, the Director, Office of Nuclear Material Safety and Safeguards, is prepared to issue an amendment to the Certificate of Compliance for the Paducah Gaseous Diffusion Plant. The staff has prepared a Compliance Evaluation Report which provides details of the staff's evaluation.

The NRC staff has determined that this amendment satisfies the criteria for a categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for this amendment.

USEC or any person whose interest may be affected may file a petition, not exceeding 30 pages, requesting review of the Director's Decision. The petition must be filed with the Commission not later than 15 days after publication of this **Federal Register** Notice. A petition for review of the Director's Decision shall set forth with particularity the interest of the petitioner and how that interest may be affected by the results of the Decision. The petition should specifically explain the reasons why review of the Decision should be permitted with particular reference to the following factors: (1) The interest of the petitioner; (2) how that interest may be affected by the Decision, including the reasons why the petitioner should be permitted a review of the Decision; and (3) the petitioner's areas of concern about the activity that is the subject matter of the Decision. Any person described in this paragraph (USEC or any person who filed a petition) may file a response to any petition for review, not to exceed 30 pages, within 10 days after filing of the petition. If no petition is received within the designated 15-day period, the Director will issue the final amendment to the Certificate of Compliance without further delay. If a petition for review is received, the Decision on the amendment application will become final in 60 days, unless the Commission grants the petition for review or otherwise acts within 60 days after publication of this **Federal Register** notice.

A petition for review must be filed with the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, by the above date.

For further details with respect to the action see: (1) The application for amendment and (2) the Commission's Compliance Evaluation Report. These items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, and at the Local Public Document Room.

Date of amendment request: March 4, 1997.

Brief description of amendment: The proposed amendment will revise the Compliance Plan and the Fundamental Nuclear Materials Control (FNMC) Plan so that they are consistent in the area of dimensional measurement calculations to determine system volumes.

Basis for finding of no significance:

1. The proposed amendment will not result in a change in the types or significant increase in the amounts of any effluents that may be released offsite.

There are no effluent releases associated with this change, the proposed changes will not affect the effluent.

2. The proposed amendment will not result in a significant increase in individual or cumulative occupational radiation exposure.

The proposed changes do not relate to controls used to minimize occupational radiation exposures, therefore, the changes will not increase exposure.

3. The proposed amendment will not result in a significant construction impact.

The proposed changes will not result in any construction, therefore, there will be no construction impacts.

4. The proposed amendment will not result in a significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents.

The proposed changes do not involve a change to any previously analyzed

accident. Therefore, the changes will not result in significant increase in the potential for, or radiological or chemical consequences from previously evaluated accidents.

5. The proposed amendment will not result in the possibility of a new or different kind of accident.

The proposed changes would not create new operating conditions or a new plant configuration that could lead to a new or different type of accident.

6. The proposed amendment will not result in a significant reduction in any margin of safety.

The use of dimensional measurement calculations to determine system volumes is the manner in which the plant has historically determined system volumes. There is no reduction in any margin of safety.

7. The proposed amendment will not result in an overall decrease in the effectiveness of the plant's safety, safeguards or security programs.

Implementation of the proposed changes will not change the safety or security programs. The proposed change to the FNMC Plan and the Compliance Plan will not decrease the effectiveness of the FNMC Plan. Use of dimensional measurement calculations to determine system volumes reflects current and approved practices. The FNMC Plan and the Compliance Plan item are being revised so that the documents will be consistent. The effectiveness of the safety, safeguards, and security programs is not decreased.

Effective date: This amendment will become effective upon signature of the Director, NMSS.

Certificate of Compliance No. GDP–1: Amendment will revise the FNMC Plan and the Compliance Plan to be consistent in the discussion of dimensional measurement calculations used to determine system volumes.

Local Public Document Room location: Paducah Public Library, 555 Washington Street, Paducah, Kentucky 42003.

Dated at Rockville, Maryland, this 12th day of June 1997.

For the Nuclear Regulatory Commission.

Carl J. Paperiello,

Director, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 97–16381 Filed 6–20–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Subcommittee Meeting on Planning and Procedures, Notice of Meeting

The ACRS Subcommittee on Planning and Procedures will hold a meeting on July 8, 1997, Room T–2B1, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance, with the exception of a portion that may be closed pursuant to 5 U.S.C. 552b(c) (2) and (6) to discuss organizational and personnel matters that relate solely to internal personnel rules and practices of ACRS, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.

The agenda for the subject meeting shall be as follows:

Tuesday, July 8, 1997—12:00 Noon until 1:30 p.m.

The Subcommittee will discuss proposed ACRS activities and related matters. It may also discuss the qualifications of candidates for appointment to the ACRS. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff person named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

Further information regarding topics to be discussed, the scheduling of sessions open to the public, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements, and the time allotted therefor can be obtained by contacting the cognizant ACRS staff person, Dr. John T. Larkins (telephone: 301/415–7360) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any changes in schedule, etc., that may have occurred.

Dated: June 17, 1997.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 97–16345 Filed 6–20–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee On Reactor Safeguards Subcommittee Meeting On Probabilistic Risk Assessment; Notice of Meeting

The ACRS Subcommittee on Probabilistic Risk Assessment will hold a meeting on July 7 and 8, 1997, Room T–2B3, 11545 Rockville Pike, Rockville, Maryland. The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Monday, July 7, 1997—8:30 a.m. until the conclusion of business.

The Subcommittee will review matters included in the Staff Requirements Memorandum dated May 27, 1997: 1) acceptance criteria for plant-specific safety goals and deriving lower-tier acceptance criteria; and 2) the use of uncertainty versus point values in the PRA-related decisionmaking process.

Tuesday, July 8, 1997—8:30 a.m. until the conclusion of business.

The Subcommittee will review the proposed Standard Review Plan (SRP) section and the associated Regulatory Guide for risk-informed, performance-based inservice inspection.

The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary