

time to be involved in NRC-licensed activities. Therefore, the public health, safety and interest require that Dr. Sadovsky be prohibited from any involvement in NRC-licensed activities for a period of one year from the date of this Order, and if he is currently involved with another licensee in NRC-licensed activities, he must immediately cease such activities, and inform the NRC of the name, address and telephone number of the employer, and provide a copy of this order to the employer. Additionally, Dr. Sadovsky is required to notify the NRC of his first employment in NRC-licensed activities following the prohibition period. Furthermore, pursuant to 10 CFR 2.202, I find that the willfulness and significance of Dr. Sadovsky's conduct described above is such that the public health, safety and interest require that this Order be immediately effective.

III

Accordingly, pursuant to sections 81, 151b, 161i, 161o, 182 and 186 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR 2.202 and 10 CFR 30.10, Part 35, and 10 CFR 150.20, it is hereby ordered, immediately effective, that:

1. For a period of one year from the date of this Order, Roy Sadovsky, D.V.M., is prohibited from engaging in NRC-licensed activities. NRC-licensed activities are those activities that are conducted pursuant to a specific or general license issued by the NRC, including, but not limited to, those activities of Agreement State licensees conducted in areas of NRC jurisdiction pursuant to the authority granted by 10 CFR 150.20.

2. For a period of one year from the date of this Order, Dr. Sadovsky shall provide a copy of this Order to any prospective employer who engages in NRC-licensed activities (as described in Section III.1 above) prior to his acceptance of employment involving non-NRC-licensed activities with such prospective employer. The purpose of this requirement is to ensure that the employer is aware of the prohibition on Dr. Sadovsky from engaging in NRC-licensed activities.

3. The first time Dr. Sadovsky is employed in NRC-licensed activities following the one year prohibition, he shall notify the Regional Administrator, NRC Region I, 475 Allendale Road, King of Prussia, Pennsylvania 19406, prior to engaging in NRC-licensed activities, including activities under an Agreement State license when activities under that license are conducted in areas of NRC jurisdiction pursuant to 10 CFR 150.20. The notice shall include the name,

address, and telephone number of the NRC or Agreement State licensee and the location where licensed activities will be performed.

The Director, Office of Enforcement, may, in writing, relax or rescind any of the above conditions upon demonstration by the Licensee of good cause.

IV

In accordance with 10 CFR 2.202, Dr. Sadovsky must, and any other person adversely affected by this Order may, submit an answer to this Order and may request a hearing on this Order, within 20 days of the date of this Order. Where good cause is shown, consideration will be given to extending the time to request a hearing. A request for extension of time must be made in writing to the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission Washington, D.C. 20555, and include a statement of good cause for the extension. The answer may consent to this Order. Unless the answer consents to this Order, the answer shall, in writing and under oath or affirmation, specifically admit or deny each allegation or charge made in this Order and shall set forth the matters of fact and law on which Dr. Sadovsky or other person adversely affected relies and the reasons as to why the Order should not have been issued. Any answer or request for a hearing shall be submitted to the Secretary, U.S. Nuclear Regulatory Commission, Attn: Chief, Docketing and Service Section, Washington, DC 20555. Copies also shall be sent to the Director, Office of Enforcement, U.S. Nuclear Regulatory Commission, Washington, DC 20555, to the Assistant General Counsel for Hearings and Enforcement at the same address, to the Regional Administrator, NRC Region I, 475 Allendale Road, King of Prussia, Pennsylvania 19406, to Dr. Sadovsky if the answer or hearing request is by a person other than Dr. Sadovsky. If a person other than Dr. Sadovsky requests a hearing, that person shall set forth with particularity the manner in which his or her interest is adversely affected by this Order and shall address the criteria set forth in 10 CFR 2.714(d).

If a hearing is requested by Dr. Sadovsky, or a person whose interest is adversely affected, the Commission will issue an Order designating the time and place of any hearing. If a hearing is held, the issue to be considered at such hearing shall be whether this Order should be sustained.

Pursuant to 10 CFR 2.202(c)(2)(i), Dr. Sadovsky may, in addition to demanding a hearing, at the time the

answer is filed or sooner, move the presiding officer to set aside the immediate effectiveness of the Order on the ground that the Order, including the need for immediate effectiveness, is not based on adequate evidence but on mere suspicion, unfounded allegations, or error.

In the absence of any request for hearing, or written approval of an extension of time in which to request a hearing, the provisions specified in Section IV above shall be final 20 days from the date of this Order without further order or proceedings. If an extension of time for requesting a hearing has been approved, the provisions specified in Section IV shall be final when the extension expires if a hearing request has not been received. An answer or a request for hearing shall not stay the immediate effectiveness of this order.

For the Nuclear Regulatory Commission.

Dated at Rockville, Maryland this 1st day of May 1997.

Edward L. Jordan,

Deputy Executive Director for Regulatory Effectiveness, Program Oversight, Investigations and Enforcement.

[FR Doc. 97-12160 Filed 5-8-97; 8:45 am]

BILLING CODE 7590-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-272 and 50-311]

Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company, Atlantic City Electric Company, Salem Nuclear Generating Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of a license amendment for Facility Operating Licenses Nos. DPR-70 and DPR-75, issued to Public Service Electric and Gas Company (PSE&G, the licensee), for operation of the Salem Nuclear Generating Station, Units 1 and 2 (Salem Units 1 and 2).

The facility consists of two pressurized-water reactors located at the licensee's site in Salem County, New Jersey.

Environmental Assessment

Identification of Proposed Action

The proposed action would change Technical Specification (TS) 3.4.3, "Relief Valves," for Salem Unit 1, and TS 3.4.5, "Relief Valves," for Salem

Unit 2, to ensure that the automatic capability of the power operated relief valves (PORVs) to relieve pressure is maintained when these valves are isolated by closure of the block valves.

The proposed action is in accordance with the licensee's application for amendment dated January 31, 1997, as supplemented by letter dated March 14, 1997.

The Need for the Proposed Action

In June of 1990, the NRC issued Generic Letter (GL) 90-06 entitled "Resolution of Generic Issue 70, 'Power-Operated Valve and Block Valve Reliability,' and Generic Issue 94 'Additional Low-Temperature Overpressurization Protection For Light-Water Reactors.'" This GL was issued to increase the reliability of the PORVs and block valves to assure that they would function as required for certain transients and accidents including Steam Generator Tube Rupture (SGTR), low temperature overpressurization protection, and plant cooldown. One of the actions required by the GL was to revise the limiting conditions for operation (LCO) of the PORVs and block valves in the TSs.

PSE&G complied by submitting a request to change the TSs, by letter NLR-N93163 dated December 8, 1993, which was incorporated in the Salem Unit 1 and 2 licenses via Amendments 150 and 130, dated April 7, 1994, respectively. The submitted request and amendments were based on the guidance provided in the GL and also later revisions that were made to the LCO under NUREG-1431, "Standard Technical Specifications Westinghouse Plants," Revision 0, dated September 1992. One of the changes afforded by NUREG-1431 was to allow PORV isolation provided the PORVs are capable of manual operation based on the mitigation of a Steam Generator Tube Rupture event; whereas, the TSs recommended in GL 90-06 addressed isolation only for valves with excessive seat leakage.

In June of 1993, Westinghouse issued Nuclear Safety Advisory letter, NSAL 93-013, which addressed the Inadvertent Safety Injection (SI) Actuation at Power event and informed plants that potential nonconservative assumptions were used in evaluating the Inadvertent SI analyses.

Westinghouse determined that crediting PORV operation could be a potential solution for the mitigation of this event. The spurious operation of the SI System at power is classified as a Condition II event, a fault of moderate frequency, as referenced in Salem's Updated Final Safety Analysis Report (UFSAR) Section

15.2.14. A Condition II event should result in a reactor shutdown with the plant being capable of returning to operation.

PSE&G has determined that an inadvertent SI at power could cause the pressurizer to become water-solid if the resulting injection of borated water is not terminated. In the event that the pressurizer becomes fully water-solid, timely PORV actuation successfully mitigates the event. However, without automatic operation of the PORVs, the Reactor Coolant System (RCS) pressure may increase to the lift setpoint of the pressurizer safety relief valves before the PORVs are manually opened. The Salem pressurizer safety valves are not designed to relieve water. It is postulated, therefore, that one or more of the valves could fail to completely reseal if relieving a water-solid pressurizer. A resulting unisolable loss of RCS inventory has been analyzed in Salem's UFSAR as a Condition III event.

A review of the current Salem TSs indicates that a TS revision is necessary to preclude the possibility of operating with PORVs that can only be cycled manually. PSE&G's re-analysis of the Inadvertent SI at Power performed to support resolution of NSAL 93-013, credits operator action to unblock the PORVs, if necessary. However, once unblocked it is unlikely that operator actions can be readily accomplished to manually cycle the PORVs such that the pressurizer safety valve pressure is not reached. Therefore, PSE&G submitted the proposed TS changes by letter dated January 31, 1997, to incorporate the results of PSE&G's analysis (i.e., to credit automatic operation of PORVs for an Inadvertent SI event), into the TSs.

Environmental Impacts of the Proposed Action

As indicated in Salem UFSAR Section 15.2.4, "Spurious Operation of The Safety Injection System at Power," the results of this transient do not lead to fuel cladding damage and thus no fission products are released. The proposed changes to the TSs assure that post transient reactor coolant system pressure relief will continue to be controllable; thus, no change in the transient result will occur. Accordingly, no changes are being made in the types of any effluent that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed

action involves features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. The principal alternative to the action would be to deny the request. Such action would not change any current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement related to the operation of Salem Nuclear Generating Station Units 1 and 2, dated April 1973.

Agencies and Persons Consulted

In accordance with its stated policy, on April 15, 1997, the staff consulted with the New Jersey State official, Mr. R. Pinney, of the New Jersey Department of Environmental Protection and Energy, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated January 31, 1997, and supplement dated March 14, 1997, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC and at the local public document room located at the Salem Free Library, 112 West Broadway, Salem, New Jersey 08079.

Dated at Rockville, Maryland, this 2nd day of May 1997.

For the Nuclear Regulatory Commission.
John F. Stolz,
*Director, Project Directorate I-2, Division of
 Reactor Projects—I/II, Office of Nuclear
 Reactor Regulation.*
 [FR Doc. 97-12148 Filed 5-8-97; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Application for a License To Import Nuclear Waste

Pursuant to 10 CFR 110.70(b) "Public notice of receipt of an application", please take notice that the Nuclear Regulatory Commission has received the following application for an import license. Copies of the application are on file in the Nuclear Regulatory Commission's Public Document Room located at 2120 L Street, N.W., Washington, D.C.

A request for a hearing or petition for leave to intervene may be filed within 30 days after publication of this notice in the **Federal Register**. Any request for hearing or petition for leave to intervene shall be served by the requestor or petitioner upon the applicant, the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; the Secretary, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; and the Executive Secretary, U.S. Department of State, Washington, D.C. 20520.

The information concerning the application follows.

NRC IMPORT LICENSE APPLICATION

Name of applicant	Date of application	Date received	Application number	Description of material			Country of origin
				Material type	Total qty	End use	
ALARON Corp.	April 18, 1997 ...	April 25, 1997 ...	IW003	Contaminated Condenser tubing.	110m ³	Decontamination and recycling.	Taiwan.

Dated this 2nd day of May 1997 at Rockville, Maryland.

For the Nuclear Regulatory Commission.
Donna C. Chaney,
*Acting Director, Division of Nonproliferation,
 Exports and Multilateral Relations, Office of
 International Programs.*
 [FR Doc. 97-12146 Filed 5-8-97; 8:45 am]
 BILLING CODE 7590-01-M

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Joint Meeting of the ACRS Subcommittees on Materials and Metallurgy and on Severe Accidents; Notice of Meeting

The ACRS Subcommittees on Materials and Metallurgy and on Severe Accidents will hold a joint meeting on June 10, 1997, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Tuesday, June 10, 1997—8:30 a.m. until the conclusion of business.

The Subcommittees will hear presentations from representatives of the NRC staff and the Nuclear Energy Institute (NEI) concerning the NRC staff approach for addressing steam generator tube integrity issues, and related matters. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as

appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittees, their consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittees, along with any of their consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittees will then hear presentations by and hold discussions with representatives of the NRC staff, NEI, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements, and the time allotted therefor can be obtained by contacting the cognizant ACRS staff engineer, Mr. Noel F. Dudley (telephone 301/415-6888) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are

urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: May 5, 1997.

Sam Duraiswamy,
Chief, Nuclear Reactors Branch.
 [FR Doc. 97-12147 Filed 5-8-97; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Instrumentation and Control Systems and Computers; Notice of Meeting

The ACRS Subcommittee on Instrumentation and Control Systems and Computers will hold a meeting on May 28-29, 1997, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, May 28, 1997—8:30 a.m. until the conclusion of business.

Thursday, May 29, 1997—8:30 a.m. until the conclusion of business.

The Subcommittee will review the proposed final Standard Review Plan (SRP) sections, Branch Technical Positions (BTPs), and Regulatory Guides associated with digital instrumentation and control systems. The Subcommittee will also review the staff's incorporation of insights from the National Academy of Sciences/ National Research Council