

petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention

and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendments requested involve no significant hazards consideration, the Commission may issue the amendments and make them immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendments requested involve a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Robert A. Capra: petitioner's name and telephone number, date petition was

mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1) (i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated September 3, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085.

Dated at Rockville, Maryland, this 5th day of September 1996.

For the Nuclear Regulatory Commission.
Donna M. Skay,

Acting Project Manager Project Directorate III-2, Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.

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[Docket Nos. 50-295 AND 50-304]

**Commonwealth Edison Company;
Notice of Consideration of Issuance of
Amendments to Facility Operating
Licenses, Proposed No Significant
Hazards Consideration Determination,
and Opportunity for a Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. DPR-39 and DPR-48 issued to Commonwealth Edison Company (ComEd, the licensee) for operation of the Zion Nuclear Power Station, Units 1 and 2, located in Lake County, Illinois.

The proposed amendments would remove the uncertainty term from the specified distance and remove the footnote which specifies the time frame it is applicable.

Before issuance of the proposed license amendments, the Commission will have made findings required by the

Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendments requested involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed changes do not involve a significant increase in the probability of occurrence or consequences of an accident previously evaluated.

The proposed change is to provide clarification as to how the F* Distance is ensured at Zion Station. The associated footnote which details the applicable timeframe is being deleted. The discussion added to the Bases section of Technical Specifications will clarify the application of eddy current uncertainty when sizing a new hard roll region and when locating indications within the F* Distance. These changes are administrative in nature and have no impact of the probability or consequences of an accident.

Application of the F* Distance to degraded steam generator tubes will not affect any of the initiators or precursors of any accident previously evaluated. Application of the proposed change will not increase the likelihood that a transient initiating event will occur because transients are initiated by equipment malfunction and/or catastrophic system failure.

As previously discussed in the ComEd submittal (ComEd letter to the NRC dated October 6, 1995, transmitting the Licensing Amendment Request Modifying the F* Distance Definition for Zion Units 1 and 2), the proposed change will allow an F* Distance of 1.05 inches to be applied to disposition steam generator tubes that are degraded in the tubesheet roll transition region. The F* Distance specifies a minimum length of tubing which must be free from any indication of degradation. Below the F* Distance, any type or size of indication, including complete circumferential through wall cracking, will not impact the structural integrity of the tube with respect to pull out forces during normal operation or accident conditions, and does not significantly affect the leakage behavior of the tube.

The Feedwater Line Break (FLB) accident was used as the limiting event in the evaluation of the F* Distance. The FLB pressure differential of 2650 psi maximizes the axial loading on the tube for pull out considerations and is bounding. In addition,

the close proximity of the tubesheet to the tube will prevent tube rupture or collapse of the tube in the tubesheet span. Because application of the F* Distance will ensure that degraded tubes will provide the same structural integrity as an original undegraded tube during normal operation and accident conditions, the probability of occurrence of an accident previously evaluated is not significantly increased.

Application of the F* Distance will not significantly increase the consequences of any accident previously evaluated. The F* Distance ensures that sufficient length of undegraded tube exists to maintain structural integrity and preclude significant leakage. Due to the proximity of the tubesheet to the tube, any leakage from degradations below the F* Distance would be negligible and would be well below the Technical Specification limits established for steam generator leakage. Tube rupture as a result of indications below the F* Distance is precluded because the tubesheet prevents outward expansion of the tube in response to internal pressure.

The relationship between the tubesheet region leak rate at the most limiting postulated accident conditions relative to that for normal plant operating conditions has been assessed. For the postulated leak source within the roll expansion, increasing the differential pressure on the tube wall increases the driving head for the leak; however, it also increases the tube to tubesheet loading.

For a leak source below the F* Distance, the maximum assumed pressure differential results in an insignificant leak rate relative to that which could be associated with normal plant operation. This is a result of the increased tube to tubesheet loading associated with the increased differential pressure. Thus for a circumferential indication within the roll expansion that is left in service in accordance with the F* Distance, any leakage under accident conditions would be less than that experienced under normal operating conditions. Therefore, any leakage under accident conditions would be less than the existing Technical Specification leakage limit, which is consistent with accident analysis assumptions.

Steam generator tube integrity must be maintained under the postulated loss of coolant accident condition of secondary-to-primary differential pressure. Based on tube collapse strength characteristics, the constraint provided to the tube by the tubesheet gives a margin between the tube collapse strength and the limiting secondary-to-primary differential pressure condition, even in the presence of circumferential or axial indications. The maximum secondary-to-primary differential pressure during a postulated LOCA is 1005 psid. This value is significantly below the residual preload between the tubes and the tube sheet. Therefore, no significant secondary-to-primary leakage would be expected to occur.

Application of the F* Distance will not affect the ability to safely shut down the operating unit and/or mitigate the consequences of an accident. Additionally, the proposed revisions will not necessitate

changes to plant operating procedures during normal operation or the emergency procedures governing accident conditions and plant recovery.

Since the proposed change only clarifies how the F* Distance is ensured at Zion Station, the proposed change will not increase the probability of occurrence or consequences of any accident previously evaluated.

2. The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes to the Technical Specifications do not involve the addition of any new or different types of safety related equipment nor do they involve the operation of any equipment required for safe operation of the facility in a manner different from those addressed in the UFSAR. No safety related equipment or function will be altered as a result of the proposed changes. Also, the procedures governing normal plant operation and recovery from an accident are not changed by the application of the F* Distance.

As previously discussed in ComEd's submittal (ComEd letter to the NRC dated October 6, 1995, transmitting the Licensing Amendment Request Modifying the F* Distance Definition for Zion Units 1 and 2), the F* Distance will allow the use of an alternate method to plugging or sleeving to repair steam generator tubes with degradation in the tubesheet region. The F* Distance ensures that the structural integrity of the steam generator tube will be equivalent to the original tube.

The proposed revised F* Distance definition is consistent with the original analysis performed to justify the acceptability of the F* Distance in dispositioning steam generator tubes with degradation in the tube sheet region. Because the size of a new hard roll joint will depend on the effective size of the mechanical roller used to install the joint, no eddy current uncertainty need be considered. For inspections following the new hard roll, the eddy current uncertainty will not be required since Zion will repair all tubes with indications in the mechanical roll region.

Therefore, it has been concluded that the proposed revision does not create the possibility of a new or different type of accident is created.

3. The proposed changes do not involve a significant reduction in a margin of safety.

Plant safety margins are established through Limiting Conditions for Operation (LCOs), limiting safety system settings, and safety limits specified in Technical Specifications. There will be no changes to the LCOs, limiting safety system settings, or the safety limits as a result of the proposed changes. Application of the F* Distance will allow degraded steam generator tubes to be repaired by an alternative method to plugging or sleeving. Steam generator tube plugging decreases the total primary reactor coolant flow rate and heat transfer capability of the steam generator. While steam generator tube sleeving only slightly reduces the reactor coolant flow rate, large numbers of sleeves can have a measurable effect on flow rate and

can complicate steam generator tube inspection activities.

Application of the F* Distance will allow a repair method that will restore the integrity of degraded steam generator tubes and will not adversely affect primary system flow rate or heat transfer capability. Application of the F* Distance will preserve the heat transfer capability of the steam generators and will maintain the design margins assumed in the analyses contained in the UFSAR. The proposed revised F* Distance definition is consistent with the original analysis performed to justify the application of the F* criteria in the dispositioning of steam generator tubes with degradation in the tube sheet region. The revision of the F* Distance is administrative in nature.

Based on the above discussion it is concluded that the proposed changes will not significantly reduce a margin of safety.

Based upon the preceding analysis it is concluded that the proposed amendment does not increase the probability of an accident previously evaluated, does not create the possibility of a new or different kind of accident from that previously evaluated, nor reduces any margin to plant safety. Therefore, this proposed amendment does not involve a significant hazards consideration as defined in 10 CFR 50.92.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendments requested involve no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendments until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendments before the expiration of the 30-day notice period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the Federal Register a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules Review and

Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By October 11, 1996, the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible

effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendments requested involve no significant hazards consideration, the Commission may issue the amendments and make them immediately effective,

notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendments requested involve a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Robert A. Capra: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1) (i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated August 16, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Waukegan Public Library, 128 N. County Street, Waukegan, Illinois 60085.

Dated at Rockville, Maryland, this 5th day of September 1996.

For the Nuclear Regulatory Commission.
Donna M. Skay,
Acting Project Manager, Project Directorate III-2, Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.
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[DOCKET NO. 50-415]

Entergy Operations, Inc.; Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-29 issued to Entergy Operations, Inc. (the licensee) for operation of Grand Gulf Nuclear Station, Unit 1 (GGNS) located in Claiborne County, Mississippi.

The proposed amendment would allow the licensee to perform the surveillance of the relief mode of operation of each of the 20 safety/relief valves (S/RVs) on the four main steam lines without physically lifting the disk of the valve off the seat at power. The proposed changes are to Surveillance Requirements (SRs) 3.4.4.3, Safety/Relief Valves, 3.5.1.7, Automatic Depressurization System Valves, and 3.6.1.6.1, Low-Low Set Valves, of the Technical Specifications, and the changes would state that the required operation of the valve to verify is that the relief-mode actuator strokes when the valve is manually actuated and the frequency of the SRs would be in accordance with the inservice testing program for the valves.

Each S/RV is a Dikkers, 8 × 10, direct-acting, spring loaded, safety valve with attached pneumatic actuator for relief-mode operation. Eight of the S/RVs use the relief mode to perform the Automatic Depressurization System (ADS) function. Also, six S/RVs, two of which are also ADS S/RVs, use the relief mode to perform the Low-Low Valve set function.

The licensee submitted an application for amendment dated May 9, 1996, as supplemented by letter August 27, 1996. A Notice of Consideration of Issuance of an Amendment to the license was issued in the Federal Register on June 19, 1996 (61 FR 31177), for the letter of May 9, 1996. The modification to the application in the letter of August 27, 1996, is to (1) state that the frequency of performing the SRs will be "in accordance with the inservice testing program" for the valves and (2) delete

the word "required" for SRs 3.5.1.7 and 3.6.1.6.1.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the request for amendment involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

a. No Significant Increase in the Probability or Consequences of an Accident Previously Evaluated Results From This Change

Each refueling outage, a test sample of the population of S/RVs is removed from the plant to perform testing as required by ASME Boiler and Pressure Vessel Code, Section XI. These S/RVs will be stroked in the relief mode during as-found testing, and are therefore verified to operate properly when each S/RV stem is raised by the relief-mode pneumatic actuator. This proposed surveillance verifies proper S/RV relief-mode operation of all installed S/RVs based upon this test sample. This testing, in conjunction with replacement of each S/RV prior to the end of its expected service life, provides reasonable assurance that the installed S/RVs will perform as well as the test population of S/RVs.

After the S/RVs have been replaced in the plant, and after all controls are reconnected, the relief-mode actuator on each newly-installed S/RV will be uncoupled from the S/RV stem, and stroked. This actuator stroke will verify that no damage has occurred to the relief-mode actuator during S/RV transportation from its storage location to its operating location. The direct coupling of the valve stem to disk provides assurance that proper relief actuation will occur when the actuator is operated. The safety-mode components are completely encased within the valve body and bonnet, which provides a rugged structure to prevent damage to these components. The remaining installed S/RVs will continue to be tested for proper control system function as previously required by Technical Specifications. The direct coupling of the S/RV stem to disk provides assurance that proper relief-mode actuation will occur when the actuator is operated. The safety mode of the GGNS S/RVs is not affected by a malfunction of the relief-mode components.

Blockage of each S/RV discharge line will be prevented by the same Foreign Material