

provisions in these part 52 licenses and regulatory approvals. If, in the future, the NRC staff seeks to impose a position in this RG in a manner that does not provide issue finality as described in the applicable issue finality provision, then the NRC staff must address the criteria for avoiding issue finality as described in the applicable issue finality provision.

Existing licensees and applicants of final design certification rules will not be required to comply with the positions set forth in this RG unless the licensee or design certification rule applicant seeks a voluntary change to its licensing basis with respect to the effects of light-water reactor coolant environments on the fatigue lives of nuclear power plant components by means of a cumulative usage factor, and where the NRC determines that the safety review of the licensee's request must include consideration of the effects of light-water reactor coolant environments on the fatigue lives of nuclear power plant components. Further information on the staff's use of the RG is contained in the RG under Section D, "Implementation."

Dated at Rockville, Maryland, this 30th day of May 2018.

For the Nuclear Regulatory Commission.

Thomas H. Boyce,

Chief, Regulatory Guidance and Generic Issues Branch, Division of Engineering, Office of Nuclear Regulatory Research.

[FR Doc. 2018-11995 Filed 6-4-18; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[NRC-2014-0023]

Effect of LWR Water Environments on the Fatigue Life of Reactor Materials

AGENCY: Nuclear Regulatory Commission.

ACTION: NUREG; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing NUREG/CR-6909, Revision 1, "Effect of LWR Water Environments on the Fatigue Life of Reactor Materials." This report summarizes the results of NRC research efforts and work performed at Argonne National Laboratory on the fatigue of piping and pressure vessel steels in light-water reactor (LWR) environments. Revision 1 of this report provides updates and improvements to the environmental fatigue correction factor approach based on an extensive update to available laboratory fatigue data from testing and results available since this

report was first published in 2007. This final document also incorporates changes to address public comments provided on the draft of Revision 1 of NUREG/CR-6909.

ADDRESSES: Please refer to Docket ID NRC-2014-0023 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Website:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2014-0023. Address questions about NRC dockets to Jennifer Borges; telephone: 301-287-9127; email: Jennifer.Borges@nrc.gov. For technical questions, contact the individuals listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.
- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document. Revision 1 of NUREG/CR-6909 is available in ADAMS under Accession No. ML16319A004.

- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT: Robert Tregoning, Office of Nuclear Regulatory Research, telephone: 301-415-2324, email: Robert.Tregoning@nrc.gov; or Appajosula Rao, Office of Nuclear Regulatory Research, telephone: 301-415-2381, email: Appajosula.Rao@nrc.gov, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001.

SUPPLEMENTARY INFORMATION:

The American Society of Mechanical Engineers Boiler and Pressure Vessel Code (Code) provides rules for the design of Class 1 components of nuclear power plants. Appendix I to Section III of the Code contains fatigue design curves for applicable structural materials. However, the effects of LWR water environments are not explicitly

addressed by the Code design curves. The existing fatigue strain-vs.-life ($\epsilon - N$) data illustrate potentially significant effects of LWR water environments on the fatigue resistance of pressure vessel and piping steels. Under certain environmental and loading conditions, fatigue lives in water relative to those in air can be significantly lower for austenitic stainless steels, nickel alloy materials, carbon steels, and low-alloy steels. In March 2007, Revision 0 of NUREG/CR-6909 (ADAMS Accession No. ML070660620) was issued. That report was the technical basis document for NRC Regulatory Guide (RG) 1.207, Revision 0, "Guidelines for Evaluating Fatigue Analyses Incorporating the Life Reduction of Metal Components Due to the Effects of the Light-Water Reactor Environment for New Reactors" (ADAMS Accession No. ML070380586). Revision 0 of NUREG/CR-6909 summarized the work performed at Argonne National Laboratory on the fatigue of piping and pressure vessel steels in LWR coolant environments. That report evaluated the existing laboratory fatigue data to identify the various materials, environmental, and loading parameters that influence fatigue crack initiation and summarized the effects of key parameters on the fatigue lives of pressure vessel and piping steels. The report presented models for estimating fatigue lives as a function of material, loading, and environmental conditions, and described the environmental fatigue correction factor for incorporating the effects of LWR coolant environments into Code fatigue evaluations.

Revision 1 of NUREG/CR-6909 provides updates and improvements to the environmental fatigue correction factor approach based on additional laboratory fatigue data and other results available since 2007. On April 17, 2014 (79 FR 21811), a draft of Revision 1 was noticed in the **Federal Register** for public comment under Docket ID NRC-2014-0023. The public comment period ended on June 2, 2014. The final version of Revision 1 of NUREG/CR-6909 reflects changes made to address the public comments. Appendix F of the document provides responses to the public comments received.

Revision 1 of NUREG/CR-6909 is the technical basis document for Revision 1 of RG 1.207, "Guidelines for Evaluating the Effects of Light-Water Reactor Water Environments in Fatigue Analyses of Metal Components" (ADAMS Accession No. ML16315A130). This RG describes methods and procedures that the NRC staff considers acceptable for use in determining the acceptable fatigue lives

of components evaluated by a cumulative usage factor calculation in accordance with the fatigue design provisions in Section III, "Rules for Construction of Nuclear Power Plant Components," of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code to account for the effects of LWR water environments. The NRC is issuing Revision 1 of RG 1.207 concurrently with Revision 1 of NUREG/CR-6909 under a separate notice associated with Docket ID NRC-2014-0244.

The NRC notes that Revision 1 of RG 1.207 was issued in draft form as a draft RG (DG-1309). The NRC published a notice of the availability of DG-1309 in the **Federal Register** on November 24, 2014 (79 FR 69884), under Docket ID NRC-2014-0244, with a public comment period that closed on January 24, 2015. Public comments on DG-1309 and the NRC staff's responses are available in ADAMS under Accession No. ML16315A127.

Dated at Rockville, Maryland, this 30th day of May 2018.

For the Nuclear Regulatory Commission.

Thomas Boyce,

Chief, Regulatory Guidance and Generic Issues Branch, Division of Engineering, Office of Nuclear Regulatory Research.

[FR Doc. 2018-11996 Filed 6-4-18; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards (ACRS); Meeting of the ACRS Subcommittee on Planning and Procedures; Notice of Meeting

The ACRS Subcommittee on Planning and Procedures will hold a meeting on June 6, 2018, 11545 Rockville Pike, Room T-2B3, Rockville, Maryland 20852.

The meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, June 6, 2018—12:00 p.m. Until 1:00 p.m.

The Subcommittee will discuss proposed ACRS activities and related matters. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Quynh Nguyen (Telephone 301-415-5844 or Email:

Quynh.Nguyen@nrc.gov) five days prior to the meeting, if possible, so that arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least thirty minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 4, 2017 (82 FR 46312). The bridgeline number for this meeting is 888-790-7128, passcode 7802533#.

Information regarding changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the DFO if such rescheduling would result in a major inconvenience.

If attending this meeting, please enter through the One White Flint North building, 11555 Rockville Pike, Rockville, Maryland 20852. After registering with Security, please contact Mr. Theron Brown at 301-415-6702 to be escorted to the meeting room.

Dated: May 23, 2018.

Mark L. Banks,

Chief, Technical Support Branch, Advisory Committee on Reactor Safeguards.

[FR Doc. 2018-12023 Filed 6-4-18; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[NRC-2018-0102]

Applications and Amendments to Facility Operating Licenses and Combined Licenses Involving Proposed No Significant Hazards Considerations and Containing Sensitive Unclassified Non-Safeguards Information and Order Imposing Procedures for Access to Sensitive Unclassified Non-Safeguards Information

AGENCY: Nuclear Regulatory Commission.

ACTION: License amendment request; notice of opportunity to comment, request a hearing, and petition for leave to intervene; order imposing procedures.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) received and is considering approval of three amendment requests. The amendment requests are for River Bend Station, Unit 1; Grand Gulf Nuclear Station, Unit 1; and Browns Ferry Nuclear Plant, Units 1, 2, and 3. For each amendment request, the NRC proposes to determine that they involve no significant hazards consideration. Because each amendment request contains sensitive unclassified non-safeguards information (SUNSI) an order imposes procedures to obtain access to SUNSI for contention preparation.

DATES: Comments must be filed by July 5, 2018. A request for a hearing must be filed by August 6, 2018. Any potential party as defined in § 2.4 of title 10 of the *Code of Federal Regulations* (10 CFR) who believes access to SUNSI is necessary to respond to this notice must request document access by June 15, 2018.

ADDRESSES: You may submit comments by any of the following methods:

- *Federal Rulemaking website:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2018-0102. Address questions about NRC dockets to Jennifer Borges; telephone: 301-287-9127; email: *Jennifer.Borges@nrc.gov*. For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *Mail comments to:* May Ma, Office of Administration, Mail Stop: TWFN-7-A60M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

For additional direction on obtaining information and submitting comments, see "Obtaining Information and Submitting Comments" in the **SUPPLEMENTARY INFORMATION** section of this document.

FOR FURTHER INFORMATION CONTACT:

Beverly Clayton, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-415-3475; email: *Beverly.Clayton@nrc.gov*.

SUPPLEMENTARY INFORMATION:

I. Obtaining Information and Submitting Comments

A. Obtaining Information

Please refer to Docket ID NRC-2018-0102, facility name, unit number(s),