Friday, February 17, 2017

9:30 a.m. Briefing on Project Aim (Public Meeting) (Contact: Tammy Bloomer: 301–415–1785).

This meeting will be webcast live at the Web address—http://www.nrc.gov/.

Week of February 20, 2017—Tentative

There are no meetings scheduled for the week of February 20, 2017.

Week of February 27, 2017—Tentative

Wednesday, March 1, 2017

10:00 a.m. Briefing on NRC International Activities (Closed Ex. 1 & 9).

Thursday, March 2, 2017

9:00 a.m. Strategic Programmatic
Overview of the Fuel Facilities and
the Nuclear Materials Users
Business Lines (Public Meeting)
(Contact: Soly Soto; 301–415–7528).
This meeting will be welcast live at

This meeting will be webcast live at the Web address—http://www.nrc.gov/.

Week of March 6, 2017—Tentative

There are no meetings scheduled for the week of March 6, 2017.

Week of March 13, 2017—Tentative

There are no meetings scheduled for the week of March 13, 2017.

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The schedule for Commission meetings is subject to change on short notice. For more information or to verify the status of meetings, contact Denise McGovern at 301–415–0981 or via email at Denise.McGovern@nrc.gov.

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The NRC Commission Meeting
Schedule can be found on the Internet
at: http://www.nrc.gov/public-involve/
public-meetings/schedule.html.

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The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (e.g., braille, large print), please notify Kimberly Meyer, NRC Disability Program Manager, at 301-287-0739, by videophone at 240-428-3217, or by email at Kimberly.Meyer-Chambers@ nrc.gov. Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

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Members of the public may request to receive this information electronically. If you would like to be added to the distribution, please contact the Nuclear Regulatory Commission, Office of the Secretary, Washington, DC 20555 (301–415–1969), or email Brenda. Akstulewicz@nrc.gov or Patricia. Jimenez@nrc.gov.

Dated: February 1, 2017.

Denise L. McGovern,

 $Policy\ Coordinator,\ Office\ of\ the\ Secretary. \\ [FR\ Doc.\ 2017-02404\ Filed\ 2-1-17;\ 4:15\ pm]$

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–461, 72–1046, 50–254, 50–265, 72–53, 50–219 and 72–15; NRC–2017–0014]

Exelon Generation Company, LLC; Clinton Power Station, Unit No. 1; Quad Cities Nuclear Power Station, Units 1 and 2; Oyster Creek Nuclear Generating Station

AGENCY: Nuclear Regulatory

Commission.

ACTION: Exemption; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption in response to an August 16, 2016, request from Exelon Generation Company, LLC (Exelon or the licensee), from certain regulatory requirements. The exemption would allow a certified fuel handler (CFH), besides a licensed senior operator, to approve the emergency suspension of security measures for Clinton Power Station, Unit No. 1 (CPS); Quad Cities Nuclear Power Station, Units 1 and 2 (QCNPS); and Oyster Creek Nuclear Generating Station (OCNGS) during certain emergency conditions or during severe weather.

DATES: The exemption was issued on January 23, 2017.

ADDRESSES: Please refer to Docket ID NRC–2017–0014 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

• Federal Rulemaking Web site: Go to http://www.regulations.gov and search for Docket ID NRC-2017-0014. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the FOR FURTHER INFORMATION CONTACT section of this document.

• NRC's Agencywide Documents Access and Management System (ADAMS): You may obtain publiclyavailable documents online in the ADAMS Public Documents collection at http://www.nrc.gov/reading-rm/adams.html. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to pdr.resource@nrc.gov. For the convenience of the reader, the ADAMS accession numbers are provided in a table in the "Availability of Documents" section of this document.

• NRC's PDR: You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT: John G. Lamb, Office of Nuclear Reactor Regulation; U.S. Nuclear Regulatory Commission, Washington DC 20555–0001; telephone: 301–415–3100; email: John.Lamb@nrc.gov.

I. Background

Exelon is the holder of Facility Operating License No. NPF-62 for CPS, Renewed Facility Operating License Nos. DPR-29 and DPR-30 for QCNPS, and Renewed Facility Operating License No. DPR-16 for OCNGS. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the NRC now or hereafter in effect. The CPS, QCNPS, and OCNGS facilities consist of boilingwater reactors located in DeWitt County, Illinois; Rock Island County, Illinois; and Ocean County, New Jersey, respectively, and site-specific licensed independent spent fuel storage installations (ISFSI) at CPS, QCNPS, and

By letter dated January 7, 2011, the licensee submitted Certification of Permanent Cessation of Operations for OCNGS. In this letter, Exelon provided notification to the NRC of its intent to permanently cease power operation no later than December 31, 2019.

By letter dated June 20, 2016, the licensee submitted Certification of Permanent Cessation of Operations for CPS. In this letter, Exelon provided notification to the NRC of its intent to permanently cease power operation by June 1, 2017.

By letter dated June 20, 2016, the licensee submitted Certification of Permanent Cessation of Operations for QCNPS. In this letter, Exelon provided notification to the NRC of its intent to permanently cease power operation by June 1, 2018.

In accordance with § 50.82(a)(1)(i) and (ii), and § 50.82(a)(2) of title 10 of the Code of Federal Regulations (10 CFR),

the 10 CFR part 50 licenses for the facilities will no longer authorize reactor operation, placement, or retention of fuel in the respective reactor vessel after certifications of permanent cessation of operations and of permanent removal of fuel from the reactor vessel are docketed for CPS, QCNPS, and OCNGS.

By letter dated September 6, 2016, the NRC approved the Certified Fuel Handler Training and Retraining Program for CPS, QCNPS, and OCNGS.

By letters dated December 14, 2016, Exelon withdrew its "Certification of Permanent Cessation of Power Operations" for CPS and QCNPS. The withdrawal letters for CPS and QCNPS did not revise its request for exemption, and did not change the effectiveness of the exemption or the conditions required to implement the actions permitted by the exemption.

II. Request/Action

On August 16, 2016, the licensee requested an exemption from § 73.55(p)(1)(i) and (ii), pursuant to § 73.5, "Specific exemptions." Section 73.55(p)(1)(i) and (ii) require, in part, that the suspension of security measures during certain emergency conditions or during severe weather be approved by a licensed senior operator. Exelon requested an exemption from these rules to allow either a licensed senior operator or a CFH to approve the suspension of security measures. There is no need for an exemption from these rules for a licensed senior operator because the current regulation allows the licensed senior operator to approve the suspension of security measures. The exemption request relates solely to the licensing requirements specified in the regulations for the staff directing suspension of security measures in accordance with $\S73.55(p)(1)(i)$ and (ii), and would allow a CFH, besides a licensed senior operator, to provide this approval. The exemption would allow the suspension of security measures during certain emergency conditions or during severe weather by a licensed senior operator or a CFH.

The current § 73.55(p)(1)(i) and (ii) regulations state the licensed senior operator can approve suspension of security measures.

The proposed exemption would authorize that the suspension of security measures must be approved as a minimum by either a licensed senior operator or a certified fuel handler, at a nuclear power plant reactor facility for which the certifications required under § 50.82(a)(1) have been submitted.

III. Discussion

The NRC's security rules have long recognized the potential need to suspend security or safeguards measures under certain conditions. Accordingly, 10 CFR 50.54(x) and (y), first published in 1983, allow a licensee to take reasonable steps in an emergency that deviate from license conditions when those steps are "needed to protect the public health and safety" and there are no conforming comparable measures (48 FR 13970; April 1, 1983). As originally issued, the deviation from license conditions must be approved by, as a minimum, a licensed senior operator. In 1986, in its final rule, "Miscellaneous Amendments Concerning the Physical Protection of Nuclear Power Plants" (51 FR 27817; August 4, 1986), the Commission issued § 73.55(a).

In 1996, the NRC made a number of regulatory changes to address decommissioning. One of the changes was to amend § 50.54 (x) and (y) to authorize a non-licensed operator called a "Certified Fuel Handler," in addition to a licensed senior operator, to approve such protective steps. Specifically, in addressing the role of the CFH during emergencies, the Commission stated in the proposed rule, "Decommissioning of Nuclear Power Reactors" (60 FR 37379; July 20, 1995):

The Commission is proposing to amend 10 CFR 50.54(y) to permit a certified fuel handler at nuclear power reactors that have permanently ceased operations and permanently removed fuel from the reactor vessel, subject to the requirements of § 50.82(a) and consistent with the proposed definition of "Certified Fuel Handler" specified in § 50.2, to make these evaluations and judgments. A nuclear power reactor that has permanently ceased operations and no longer has fuel in the reactor vessel does not require a licensed individual to monitor core conditions. A certified fuel handler at a permanently shutdown and defueled nuclear power reactor undergoing decommissioning is an individual who has the requisite knowledge and experience to evaluate plant conditions and make these judgments.

In the final rule (61 FR 39298; July 29, 1996), the NRC added the following definition to § 50.2: "Certified fuel handler means, for a nuclear power reactor facility, a non-licensed operator who has qualified in accordance with a fuel handler training program approved by the Commission." However, the decommissioning rule did not propose or make parallel changes to § 73.55(a), and did not discuss the role of a non-licensed CFH.

In the final rule, "Power Reactor Security Requirements" (74 FR 13926; March 27, 2009), the NRC relocated the security suspension requirements from § 73.55(a) to § 73.55(p)(1)(i) and (ii). The role of a CFH was not discussed in the rulemaking, so the suspension of security measures in accordance with § 73.55(p) continued to require approval as a minimum by a licensed senior operator, even for a site that otherwise no longer operates.

However, pursuant to § 73.5, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR part 73, as it determines are authorized by law and will not endanger life or property or the common defense and security, and are otherwise in the public interest.

A. The Exemption Is Authorized by Law

The exemption from § 73.55(p)(1)(i) and (ii) would allow a CFH, besides a licensed senior operator, to approve the suspension of security measures, under certain emergency conditions or severe weather. The licensee intends to align these regulations with § 50.54(y) by using the authority of a CFH in place of a licensed senior operator to approve the suspension of security measures during certain emergency conditions or during severe weather.

Per § 73.5, the Commission is allowed to grant exemptions from the regulations in 10 CFR part 73, as authorized by law. The NRC staff has determined that granting of the licensee's proposed exemption will not result in a violation of the Atomic Energy Act of 1954, as amended, or other laws. Therefore, the exemption is authorized by law.

B. Will Not Endanger Life or Property or the Common Defense and Security

Relaxing the requirement to allow a CFH, besides a licensed senior operator, to approve suspension of security measures during emergencies or severe weather will not endanger life or property or the common defense and security for the reasons described in this section.

First, § 73.55(p)(2) continues to require that "[s]uspended security measures must be reinstated as soon as conditions permit."

Second, the suspension for non-weather emergency conditions under § 73.55(p)(1)(i) will continue to be invoked only "when this action is immediately needed to protect the public health and safety and no action consistent with license conditions and technical specifications that can provide adequate or equivalent protection is immediately apparent." Thus, the exemption would not prevent the licensee from meeting the underlying purpose of § 73.55(p)(1)(i) to protect

public health and safety even after the exemption is granted.

Third, the suspension for severe weather under $\S73.55(p)(1)(ii)$ will continue to be used only when "the suspension of affected security measures is immediately needed to protect the personal health and safety of security force personnel and no other immediately apparent action consistent with the license conditions and technical specifications can provide adequate or equivalent protection." The requirement to receive input from the security supervisor or manager will remain. The exemption would not prevent the licensee from meeting the underlying purpose of § 73.55(p)(1)(ii) to protect the health and safety of the security force.

Additionally, by letter dated September 6, 2016, the NRC approved Exelon's CFH training and retraining program for the CPS, QCNPS, and OCNGS facilities. The NRC staff found that, among other things, the program addresses the safe conduct of decommissioning activities, safe handling and storage of spent fuel, and the appropriate response to plant emergencies. Because the CFH is sufficiently trained and qualified under an NRC-approved program, the NRC staff considers a CFH to have sufficient knowledge of operational and safety concerns, such that allowing a CFH to suspend security measures during emergencies or severe weather will not result in undue risk to public health and safety

In addition, the exemption does not reduce the overall effectiveness of the physical security plan and has no adverse impacts to Exelon's ability to physically secure the sites or protect special nuclear material at CPS, QCNPS, and OCNGS, and thus would not have an effect on the common defense and security. The NRC staff has concluded that the exemption would not reduce security measures currently in place to protect against radiological sabotage. Therefore, relaxing the requirement to allow a CFH, besides a licensed senior operator, to approve the suspension of security measures in an emergency or during severe weather, does not adversely affect public health and safety issues or the assurance of the common defense and security.

C. Is Otherwise in the Public Interest

Exelon's proposed exemption would relax the requirement to allow a CFH, besides a licensed senior operator, to approve suspension of security measures in an emergency when "immediately needed to protect the public health and safety" or during

severe weather when "immediately needed to protect the personal health and safety of security force personnel." Without the exemption, the licensee cannot implement changes to its security plan to authorize a CFH to approve the temporary suspension of security regulations during an emergency or severe weather, comparable to the authority given to the CFH by the NRC when it published § CFR 50.54(y). Instead, the regulations would continue to require that a licensed senior operator be available to make decisions for a permanently shutdown plant, even though CPS, QCNPS, and OCNGS would no longer require a licensed senior operator after the certifications required by 10 CFR 50.82(a)(1)(i) and 10 CFR 50.82(a)(1)(ii) were submitted. It is unclear how the licensee would implement emergency or severe weather suspensions of security measures without a licensed senior operator. This exemption is in the public interest for two reasons. First, without the exemption, there is uncertainty on how the licensee will invoke temporary suspension of security matters that may be needed for protecting public health and safety or the safety of the security force during emergencies and severe weather. The exemption would allow the licensee to make decisions pursuant to § 73.55(p)(1)(i) and (ii) without having to maintain a staff of licensed senior operators. The exemption would also allow the licensee to have an established procedure in place to allow a trained CFH to suspend security measures in the event of an emergency or severe weather. Second, the consistent and efficient regulation of nuclear power plants serves the public interest. This exemption would assure consistency between the security regulations in 10 CFR part 73 and CFR 50.54(y), and the requirements concerning licensed operators in 10 CFR part 55. The NRC staff has determined that granting the licensee's proposed exemption would allow the licensee to designate an alternative position, with qualifications appropriate for a permanently shutdown and defueled reactor, to approve the suspension of security measures during an emergency to protect the public health and safety, and during severe weather to protect the safety of the security force, consistent with the similar authority provided by § 50.54(y). Therefore, the exemption is in the public interest.

D. Environmental Considerations

The NRC's approval of the exemption to security requirements belongs to a category of actions that the Commission, by rule or regulation, has declared to be a categorical exclusion, after first finding that the category of actions does not individually or cumulatively have a significant effect on the human environment. Specifically, the exemption is categorically excluded from further analysis under § 51.22(c)(25).

Under § 51.22(c)(25), the granting of an exemption from the requirements of any regulation of Chapter I to 10 CFR is a categorical exclusion provided that (i) there is no significant hazards consideration; (ii) there is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite; (iii) there is no significant increase in individual or cumulative public or occupational radiation exposure; (iv) there is no significant construction impact; (v) there is no significant increase in the potential for or consequences from radiological accidents; and (vi) the requirements from which an exemption is sought involve: Safeguard plans, and materials control and accounting inventory scheduling requirements; or involve other requirements of an administrative, managerial, or organizational nature.

The Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation, has determined that approval of the exemption request involves no significant hazards consideration because allowing a CFH, besides a licensed senior operator, to approve the security suspension at a defueled shutdown power plant does not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The exempted security regulation is unrelated to any operational restriction. Accordingly, there is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite; and no significant increase in individual or cumulative public or occupational radiation exposure. The exempted regulation is not associated with construction, so there is no significant construction impact. The exempted regulation does not concern the source term (i.e., potential amount of radiation in an accident), nor mitigation. Thus, there is no significant increase in the potential for, or consequences of, a radiological accident. The requirement to have a licensed senior operator approve departure from security actions may be

viewed as involving either safeguards, materials control, or managerial matters.

Therefore, pursuant to § 51.22(b) and (c)(25), no environmental impact statement or environmental assessment need be prepared in connection with the approval of this exemption request.

IV. Conclusions

Accordingly, the Commission has determined that, pursuant to 10 CFR

73.5, the exemption is authorized by law and will not endanger life or property or the common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants the licensee's request for an exemption from the requirements of 10 CFR 73.55(p)(1)(i) and (ii), to authorize that the suspension of security measures must be approved as a minimum by either a licensed senior

operator or a certified fuel handler, at a nuclear power plant reactor facility for which the certifications required under 10 CFR 50.82(a)(1) have been submitted.

The exemption is effective upon receipt.

V. Availability of Documents

The documents identified in the following table are available to interested persons.

Title	Date	ADAMS accession No.
Exelon letter to NRC, "Permanent Cessation of Operations at Oyster Creek Nuclear Generating Station."	1/07/2011	ML110070507
Exelon letter to NRC, Clinton Power Station, Unit 1, "Certification of Permanent Cessation of Power Operations."	6/20/2016	ML16172A137
Exelon letter to NRC, Quad Cities Nuclear Power Station, Units 1 and 2, "Certification of Permanent Cessation of Power Operations."	6/20/2016	ML16172A151
NRC letter to Exelon, Oyster Creek Nuclear Generating Station; "Clinton Power Station, Unit No. 1; and Quad Cities Nuclear Power Station, Units 1 and 2—Approval of Certified Fuel Handler Training and Re-	9/06/2016	ML16222A787
training Program."		
Exelon letter to NRC, Clinton Power Station, Unit No. 1, Quad Cities Nuclear Power Station, Units 1 and 2, and Oyster Creek Nuclear Generating Station, "Request for Exemption from Specific Provisions in 10 CFR 7355(p)(1)(i) and (p)(1)(ii) Related to the Suspension of Security Measures in an Emergency or		
During Severe Weather."	8/16/2016	ML16229A133
Exelon letter to NRC, Oyster Creek Nuclear Generating Station, "License Amendment Request—Proposed Changes to Technical Specifications Section 6.0 Administrative Controls for Permanently Defueled Con-	-/	
dition" Exelon letter to NRC, Clinton Power Station, Unit No. 1, "License Amendment Request—Proposed	5/17/2016	ML16138A129
Changes to Technical Specifications Section 5.0 Administrative Controls for Permanently Defueled Condition"	7/28/2016	ML16210A300
Exelon letter to NRC, Quad Cities Nuclear Power Station, Units 1 and 2, "License Amendment Request—		
Proposed Changes to Technical Specifications Section 5.0 Administrative Controls for Permanently Defueled Condition	10/20/2016	ML16294A203
Exelon Letter to NRC, Quad Cities Nuclear Power Station, Units 1 and 2, "Withdrawal of Certification of		
Permanent Cessation of Power Operations."	12/14/2016	ML16349A311
sation of Power Operations."	12/14/2016	ML16349A314

Dated at Rockville, Maryland, this 23rd day of January 2017.

For the Nuclear Regulatory Commission. **Anne T. Boland**,

Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2017–02336 Filed 2–2–17; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC-2017-0016]

Guidance for Developing Principal Design Criteria for Non-Light Water Reactors

AGENCY: Nuclear Regulatory Commission.

ACTION: Draft regulatory guide; request for comment.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing for public comment draft regulatory guide (DG), DG–1330, "Guidance for Developing Principal Design Criteria for Non-Light

Water Reactors." This DG is a proposed new regulatory guide (RG) to provide designers, applicants, and licensees of non-light water cooled nuclear reactors (non-LWR) guidance for developing principal design criteria (PDC) for a proposed facility. The PDC establish the necessary design, fabrication, construction, testing, and performance requirements for structures, systems, and components important to safety; that is, structures, systems, and components that provide reasonable assurance that the facility can be operated without undue risk to the health and safety of the public.

DATES: Submit comments by April 4, 2017. Comments received after this date will be considered if it is practical to do so, but the NRC is able to ensure consideration only for comments received on or before this date. Although a time limit is given, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

ADDRESSES: You may submit comments by any of the following methods:

- Federal Rulemaking Web site: Go to http://www.regulations.gov and search for Docket ID NRC-2017-0016. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individuals listed in the FOR FURTHER INFORMATION CONTACT section of this document.
- *Mail comments to:* Cindy Bladey, Office of Administration, Mail Stop: OWFN-12-H08, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

For additional direction on obtaining information and submitting comments, see "Obtaining Information and Submitting Comments" in the SUPPLEMENTARY INFORMATION section of this document.

FOR FURTHER INFORMATION CONTACT: Jan Mazza, Office of New Reactors, telephone: 301–415–0498, email: Jan.Mazza@nrc.gov, or Mark Orr, Office of Nuclear Regulatory Research, telephone: 301–415–6003, email: