

contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in major inconvenience.

Dated: October 19, 2009.

**Antonio Dias,**

*Chief, Reactor Safety Branch B, Advisory Committee on Reactor Safeguards.*

[FR Doc. E9-25657 Filed 10-23-09; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

### Advisory Committee on Reactor Safeguards (ACRS); Meeting of the ACRS Subcommittee on AP1000; Notice of Meeting

The ACRS Subcommittee on the AP1000 will hold a meeting on November 19–20, 2009, 11545 Rockville Pike, Room T2–B3, Rockville, Maryland.

The meeting will be open to public attendance.

The proposed agenda for the subject meeting is as follows:

Thursday, November 19, 2009—8:30 a.m.–5 p.m.

Friday, November 20, 2009—8:30 a.m.–5 p.m.

The Subcommittee will review selected chapters of the Draft Safety Evaluation Report associated with the amendment to the Westinghouse AP1000 Design Certification Document. The Subcommittee will hear presentations by and hold discussions with both Westinghouse and NRC staff representatives regarding this matter. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as appropriate, for deliberation by the Full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Mr. Michael Lee, (Telephone 301–415–6887, E-mail: [Mike.Lee@nrc.gov](mailto:Mike.Lee@nrc.gov)) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be e-mailed to the DFO one day before meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a compact disk containing each presentation at

least 30 minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 14, 2009 (74 FR 58268–58269).

Detailed ACRS meeting agendas and meeting transcripts are available on the NRC Web site at <http://www.nrc.gov/reading-rm/doc-collections/acrs/>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the Web site cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in major inconvenience.

Dated: October 19, 2009.

**Antonio F. Dias,**

*Chief, Reactor Safety Branch B, Advisory Committee on Reactor Safeguards.*

[FR Doc. E9-25730 Filed 10-23-09; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

### Advisory Committee on Reactor Safeguards (ACRS) Meeting of The ACRS Subcommittee on Reliability and Probabilistic Risk Assessment; Notice of Meeting

The ACRS Subcommittee on Reliability and Probabilistic Risk Assessment (PRA) will hold a meeting on November 12, 2009, Room T2–B3, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Thursday, November 12, 2009—8:30 a.m.–5 p.m.

The Subcommittee will review NRC's proposed policy statement on safety culture and experience with treatment of safety culture in the reactor oversight process. The Subcommittee will hear presentations by and hold discussions with representatives of the NRC staff and other interested persons regarding these matters. The Subcommittee will gather information, analyze relevant issues and facts, and formulate proposed positions and actions, as

appropriate, for deliberation by the full Committee.

Members of the public desiring to provide oral statements and/or written comments should notify the Designated Federal Official (DFO), Dr. John H. Flack (Telephone: 301–415–0426, E-mail: [John.Flack@nrc.gov](mailto:John.Flack@nrc.gov)) five days prior to the meeting, if possible, so that appropriate arrangements can be made. Thirty-five hard copies of each presentation or handout should be provided to the DFO thirty minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the DFO one day before meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the DFO with a CD containing each presentation at least 30 minutes before the meeting. Electronic recordings will be permitted only during those portions of the meeting that are open to the public. Detailed procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 14, 2009, (74 FR 58268–58269).

Detailed meeting agendas and meeting transcripts are available on the NRC Web site at <http://www.nrc.gov/reading-rm/doc-collections/acrs/>. Information regarding topics to be discussed, changes to the agenda, whether the meeting has been canceled or rescheduled, and the time allotted to present oral statements can be obtained from the Web site cited above or by contacting the identified DFO. Moreover, in view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with these references if such rescheduling would result in major inconvenience.

Dated: October 19, 2009.

**Cayetano Santos,**

*Chief, Reactor Safety Branch A, Advisory Committee on Reactor Safeguards.*

[FR Doc. E9-25729 Filed 10-23-09; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[Docket No. 70-7005; NRC-2009-0283]

### In the Matter of Waste Control Specialists, LLC; Order Modifying Exemption

**AGENCY:** U.S. Nuclear Regulatory Commission.

**ACTION:** Issuance of Order To Modify Waste Control Specialists, LLC's

Exemption from Requirements of 10 CFR part 70.

**FOR FURTHER INFORMATION CONTACT:**

Nishka Devaser, Environmental Protection and Performance Assessment Directorate, Division of Waste Management and Environmental Protection, Office of Federal and State Materials and Environmental Management Programs, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Telephone: (301) 415-5196, fax number: (301) 415-5397; e-mail: [Nishka.Devaser@nrc.gov](mailto:Nishka.Devaser@nrc.gov).

**SUPPLEMENTARY INFORMATION:**

**I. Introduction**

Pursuant to 10 CFR 2.106, the Nuclear Regulatory Commission (NRC) is providing notice in the Matter of Waste Control Specialists, LLC (WCS) of the issuance of an order to modify WCS's exemption from the requirements of 10 CFR part 70.

**II. Further Information**

**I.**

WCS operates a facility in Andrews County, Texas, that is currently licensed to process and store certain types of low-level waste (LLW) and mixed waste (MW), and dispose of hazardous and toxic waste. Texas is an Agreement State. On November 30, 1997, this facility was licensed by the State of Texas Department of Health (TDH) under a 10 CFR part 30 equivalent radioactive materials license to possess, treat, and store LLW (R04971). License R04971 is currently under the jurisdiction of the Texas Commission on Environmental Quality (TCEQ). The facility is also licensed by the TCEQ to treat and dispose of hazardous waste. In 1997, WCS began accepting Resource Conservation and Recovery Act (RCRA)

and Toxic Substance Control Act (TSCA) wastes for treatment, storage, and disposal. Later that year, WCS received a license from TDH for treatment and storage of MW and LLW. The MW and LLW streams may contain quantities of special nuclear material (SNM). On May 29, 2008, the TCEQ issued a license to WCS that authorizes WCS to receive and dispose of byproduct material as defined in Title 25 of the Texas Administrative Code, Section 289.260(c)(4). On January 14, 2009, the TCEQ denied hearing requests and issued an order which allows a license to be granted for disposal of LLW after the applicant demonstrates ownership of all mineral rights. The order provides that a license may not be issued, signed or granted until such demonstration is made.

Section 70.3 of 10 CFR part 70 requires persons who own, acquire, deliver, receive, possess, use, or transfer SNM to obtain a license pursuant to the requirements of 10 CFR part 70. The licensing requirements in 10 CFR part 70 apply to persons in Agreement States possessing greater than critical mass quantities as defined in 10 CFR 150.11. However, pursuant to 10 CFR 70.17(a), "the Commission may \* \* \* grant such exemptions from the requirements of the regulations in this part as it determines are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest."

In September 2000, WCS requested an exemption from the licensing requirements in 10 CFR part 70. On November 21, 2001, the NRC transmitted an Order to WCS granting an exemption to WCS from certain NRC regulations and permitted WCS, under specified conditions, to possess waste containing SNM in greater quantities

than specified in 10 CFR part 150, at WCS's storage and treatment facility in Andrews County, Texas, without obtaining an NRC license pursuant to 10 CFR part 70. The NRC exemption applies only to activities authorized by TCEQ License R04971. The Order was published in the **Federal Register** on November 15, 2001 (66 FR 57489). The conditions specified in the Order are discussed in the November 2001 Safety Evaluation Report (SER) that supported the 2001 Order.

By letters dated August 6, 2003, and March 14, 2004, Waste Control Specialists LLC (WCS) requested an amendment to its exemption, which would allow it to use additional reagents for chemical stabilization of mixed waste containing SNM. The NRC transmitted the revised Order to WCS on November 4, 2004. The Order was published in the **Federal Register** on November 12, 2004 (69 FR 65468). The modified conditions specified in the Order are discussed in the August 2004 Safety Evaluation Report (SER) that supported the 2004 Order.

In a letter dated December 10, 2007, WCS requested additional modifications to its exemption from certain NRC regulations relative to the possession of SNM that is authorized by its TCEQ License R04971. By letter dated January 22, 2008, NRC acknowledged WCS' request.

WCS' letter dated December 10, 2007, and NRC's acknowledgement dated January 22, 2008, are available at NRC's Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html>. NRC's Agencywide Document Access and Management System (ADAMS) is available at this Web site. The ADAMS accession numbers for the December 10, 2007, and January 22, 2008, letters are:

Document description	Accession No.
December 10, 2007, WCS request for modification of the Order .....	ML073550638
January 22, 2008, NRC acknowledgement of WCS request .....	ML080150622

**II.**

The NRC staff considers that the appropriate action is to grant WCS's exemption request, with additional modifications. Currently, WCS is exempted from the requirements of 10 CFR Part 70, including the requirements for an NRC license in 10 CFR 70.3, for activities authorized by TCEQ License R04971. This modification specifically would allow WCS to: Discontinue confirmation sampling upon receipt of waste that WCS verifies is adequately

characterized by a generator to be uniform and which contains less than one-tenth of the SNM concentration limits presented in Condition 1; and to discontinue confirmatory sampling requirements of Condition 7 of the Order for sealed sources. By letter dated January 22, 2008, NRC informed WCS that it would clarify Condition 2, which states that waste must not contain "pure forms" of chemicals containing carbon, fluorine, magnesium, or bismuth in bulk quantities. NRC is also clarifying

requirements for spatial uniformity of SNM concentrations in waste. The NRC is also revising Condition 4 of the Order, which currently limits the amount of highly water soluble SNM in each package, to address security concerns raised by the NRC staff during its review. Therefore, WCS's exemption is modified as follows:

1. Concentrations of SNM in individual waste containers and/or during processing shall not exceed the following values:

SNM Isotope	Operational limit (gram SNM/gram waste)	Measurement uncertainty (gram SNM/gram waste)
U-233 .....	4.7E-04	7.1E-05
U-235 (10 percent enriched) .....	9.9E-04	1.5E-04
U-235 (100 percent enriched) .....	6.2E-04	9.3E-05
Pu-239 .....	2.8E-04	4.2E-05
Pu-241 .....	2.2E-04	3.2E-05

When mixtures of these SNM isotopes are present in the waste, the sum-of-the-fractions rule, as illustrated below, shall be used.

$$\frac{\text{U-233 conc}}{\text{U-233 limit}} + \frac{100\text{wt}\% \text{U-235 conc}}{100\text{wt}\% \text{U-235 limit}} + \frac{10\text{wt}\% \text{U-235 conc}}{10\text{wt}\% \text{U-235 limit}} + \frac{\text{Pu-239 conc}}{\text{Pu-239 limit}} + \frac{\text{Pu-241 conc}}{\text{Pu-241 limit}} \leq 1$$

The measurement uncertainty values in column 3 above represent the maximum one-sigma uncertainty associated with the measurement of the

concentration of the particular radionuclide.

The SNM must be uniformly distributed throughout the waste, such that the limiting concentrations must

not be exceeded on average in any contiguous mass of 600 kilograms.

2. The mass concentration of carbon, fluorine, and bismuth in the waste must be limited as follows:

SNM Isotope	Carbon	Fluorine	Bismuth
U-233 .....	28 wt% .....	34 wt% .....	34 wt%.
U-235(10) .....	25 wt% .....	35 wt% .....	31 wt%.
U-235(100) .....	41 wt% .....	42 wt% .....	33 wt%.
Pu-239 .....	43 wt% .....	43 wt% .....	34 wt%.
Pu-241 .....	37 wt% .....	39 wt% .....	32 wt%.

For waste containing mixtures of C, F, and Bi, the sum of the weight fractions of C, F, and Bi shall be compared to the most restrictive maximum allowable weight fractions for any one of those elements. Similarly, where mixtures of radionuclides are present in the waste, the limiting maximum allowable weight fraction of C, F, and Bi shall be applied.

The presence of the above materials will be determined and documented by the generator, based on process knowledge or testing.

3. Waste accepted shall not contain total quantities of beryllium, hydrogenous material enriched in deuterium, or graphite above one tenth of one percent of the total weight of the waste. The presence of the above materials will be determined and documented by the generator, based on process knowledge, or testing.

4. Possession of highly water soluble forms of SNM shall not exceed the amount of SNM of low strategic significance defined in 10 CFR 73.2. Highly soluble forms of SNM include, but are not limited to: uranium sulfate, uranyl acetate, uranyl chloride, uranyl formate, uranyl fluoride, uranyl nitrate, uranyl potassium carbonate, uranyl sulfate, plutonium chloride, plutonium fluoride, and plutonium nitrate. The presence of the above materials will be

determined and documented by the generator, based on process knowledge or testing.

5. Processing of mixed waste containing SNM will be limited to chemical stabilization (i.e., mixing waste with reagents). For batches with more than 600 kilograms of waste, the total mass of SNM shall not exceed the concentration limits in Condition 1 times 600 kilograms of waste.

6. Prior to shipment of waste, WCS shall require generators to provide a written certification containing the following information for each waste stream:

a. Waste Description. The description must detail how the waste was generated, list the physical forms in the waste, and identify uranium chemical composition.

b. Waste Characterization Summary. The data must include a general description of how the waste was characterized (including the volumetric extent of the waste, and the number, location, type, and results of any analytical testing), the range of SNM concentrations, and the analytical results with error values used to develop the concentration ranges.

c. Uniformity Description. A description of the process by which the waste was generated showing that the

spatial distribution of SNM is homogeneous or other information supporting spatial homogeneity.

d. Manifest Concentration. The generator must describe the methods to be used to determine the concentrations on the manifests. These methods could include direct measurement and the use of scaling factors. The generator must describe the uncertainty associated with sampling and testing used to obtain the manifest concentrations.

WCS shall review the above information and, if adequate, approve in writing this pre-shipment waste characterization and assurance plan before permitting the shipment of a waste stream. This will include statements that WCS has a written copy of all the information required above, that the characterization information is adequate and consistent with the waste description, and that the information is sufficient to demonstrate compliance with Conditions 1 through 4. Where generator process knowledge is used to demonstrate compliance with Conditions 1, 2, 3, or 4, WCS shall review this information and determine when testing is required to provide additional information in assuring compliance with the Conditions. WCS shall retain this information as required

by the State of Texas to permit independent review.

At the time waste is received, WCS shall require generators of SNM waste to provide a written certification with each waste manifest that states that the SNM concentrations reported on the manifest do not exceed the limits in Condition 1, and that the waste meets Conditions 2 through 4.

WCS shall require generators to sample and determine the SNM concentration for each waste stream, not to include sealed sources, at a frequency of once per 600 kg if the concentrations are above one tenth the SNM limits of Condition 1. The measurement uncertainty shall not exceed the uncertainty value in Condition 1 and shall be provided on the written certification.

7. WCS shall sample and determine the SNM concentration for each waste stream, not to include sealed sources, at a frequency of once per 600 kg if the concentrations are above one tenth the SNM limits of Condition 1. This confirmatory testing is not required for waste to be disposed of at DOE's WIPP facility.

8. WCS shall notify the NRC, Region IV office within 24 hours if any of the above Conditions are violated. A written notification of the event must be provided within 7 days.

9. WCS shall obtain NRC approval prior to changing any activities associated with the above Conditions.

### III.

Based on the staff's evaluation, the Commission has determined, pursuant to 10 CFR 70.17(a), that the exemption as described above at the WCS facility is authorized by law, will not endanger life or property or the common defense and security and is otherwise in the public interest. Accordingly, by this Order, the Commission hereby grants this exemption subject to the above conditions. The exemption will become effective after the State of Texas has incorporated the above conditions into WCS's RML.

Pursuant to the requirements in 10 CFR part 51, the Commission has published an Environmental Assessment for the proposed action wherein it has determined that the granting of this exemption will have no significant impacts on the quality of the human environment. This finding was noticed in the **Federal Register** on October 15, 2009 (74 FR 52981–52985).

Dated at Rockville, Maryland this 20th day of October 2009.

For the U.S. Nuclear Regulatory Commission.

**Larry W. Camper,**

*Division Director, Division of Waste Management and Environmental Protection, Office of Federal and State Materials and Environmental Management Programs.*

[FR Doc. E9–25662 Filed 10–23–09; 8:45 am]

**BILLING CODE 7590–01–P**

## NUCLEAR REGULATORY COMMISSION

**[NRC–2009–0465]**

### Withdrawal of Regulatory Guides 4.5 and 4.6

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Withdrawal of Regulatory Guides 4.5, “Measurements of Radionuclides in the Environment—Sampling and Analysis of Plutonium in Soil” and 4.6, “Measurements of Radionuclides in the Environment—Strontium-89 and Strontium-90 Analysis.”

#### FOR FURTHER INFORMATION CONTACT:

George Powers, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, *telephone:* 301–251–7449 or e-mail *George.Powers@nrc.gov*.

#### SUPPLEMENTARY INFORMATION:

#### I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is withdrawing Regulatory Guide (RG) 4.5, “Measurements of Radionuclides in the Environment—Sampling and Analysis of Plutonium in Soil,” and RG 4.6, “Measurements of Radionuclides in the Environment—Strontium-89 and Strontium-90 Analysis.” Both of these guides were published in May 1974.

These regulatory guides provide prescriptive guidance to licensees and applicants on the sampling and laboratory analysis of Strontium and Plutonium. The guides provided guidance on compliance with a provision in 10 CFR Part 20, “Standards for Protection against Radiation.” That provision, 10 CFR 20.106, “Concentrations in Effluents to Unrestricted Areas,” was deleted, and that subject matter is addressed in a new section, 10 CFR 20.1302.

“Compliance with dose limits for individual members of the public.” Paragraphs (a) and (b) of 20.1302 contain the effluent standards and allowable radionuclide concentrations in effluent releases.

Updated performance based guidance for the measurement of plutonium (Pu), strontium-89 (Sr<sup>89</sup>), and strontium-90

(Sr<sup>90</sup>) is now provided through Regulatory Guide 4.15, “Quality Assurance for Radiological Monitoring Programs (Inception through Normal Operations to License Termination)—Effluent Streams and the Environment,” published July 2007. This guidance references NUREG–1576, “Multi-Agency Radiological Laboratory Analytical Protocols Manual (MARLAP),” published in July 2004. MARLAP provides analytical detail for measurement of Pu, Sr<sup>89</sup>, and Sr<sup>90</sup> which includes methods described in RG 4.5 and 4.6 and more recent methods and procedures that are also acceptable to the staff. NUREG–1576 is available electronically through the NRC's Agencywide Documents Access and Management System at <http://www.nrc.gov/reading-rm/adams.html>, under Accession No. ML060930645.

### II. Further Information

The withdrawal of RGs 4.5 and 4.6 does not alter any prior or existing licensing commitments or conditions based on their use. The guidance provided in these regulatory guides is neither necessary nor current. Regulatory guides may be withdrawn when their guidance is superseded by congressional action or no longer provides useful information.

Regulatory guides are available for inspection or downloading through the NRC's public Web site under “Regulatory Guides” in the NRC's Electronic Reading Room at: <http://www.nrc.gov/reading-rm/doc-collections>.

In addition, regulatory guides are also available for inspection at the NRC's Public Document Room (PDR), Room O–1 F21, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852–2738. The PDR's mailing address is US NRC PDR, Washington, DC 20555–0001. You can reach the PDR staff by telephone at 301–415–4737 or 800–397–4209, by fax at 301–415–3548, and by e-mail to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

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Dated at Rockville, Maryland, this 13th day of October, 2009.

For the Nuclear Regulatory Commission.

**Andrea D. Valentin,**

*Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.*

[FR Doc. E9–25659 Filed 10–23–09; 8:45 am]

**BILLING CODE 7590–01–P**