

intervene, and have the opportunity to participate fully in the conduct of the hearing.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held. If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment. If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Nontimely requests and/or petitions and contentions will not be entertained absent a determination by the Commission or the presiding officer of the Atomic Safety and Licensing Board that the petition, request and/or the contentions should be granted based on a balancing of the factors specified in 10 CFR 2.309(c)(1)(i)-(viii).

A request for a hearing or a petition for leave to intervene must be filed by: (1) First class mail addressed to the Office of the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemaking and Adjudications Staff; (2) courier, express mail, and expedited delivery services: Office of the Secretary, Sixteenth Floor, One White Flint North, 11555 Rockville Pike, Rockville, Maryland, 20852, Attention: Rulemaking and Adjudications Staff; (3) e-mail addressed to the Office of the Secretary, U.S. Nuclear Regulatory Commission, HEARINGDOCKET@NRC.GOV; or (4) facsimile transmission addressed to the Office of the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC, Attention: Rulemakings and Adjudications Staff at (301) 415-1101, verification number is (301) 415-1966. A copy of the request for hearing and petition for leave to intervene should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and it is requested that copies be transmitted either by means of facsimile transmission to 301-415-3725 or by e-mail to OGCMailCenter@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to the Mr. Bradley J. Fewell, Associate General Counsel, Exelon Generation Company, LLC, 4300

Winfield Road, Warrenton, IL 60555, attorney for the licensee.

For further details with respect to this action, see the application for amendment dated July 10, 2007, which is available for public inspection at the Commission's PDR, located at One White Flint North, File Public Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management System's (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 29th day of August 2007.

For the Nuclear Regulatory Commission.

Christopher Gratton, Sr.,

Project Manager, Plant Licensing Branch III-2, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. E7-17493 Filed 9-4-07; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-298]

Nebraska Public Power District; Cooper Nuclear Station; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment to Facility Operating License No. DPR-46, issued to Nebraska Public Power District (NPPD, the licensee), for operation of the Cooper Nuclear Power Station (CNS) located in Nemaha County, Nebraska. Therefore, as specified in Title 10 of the *Code of Federal Regulations* (10 CFR) section 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action is in response to the licensee's application dated October 17, 2006, as supplemented by letters dated February 7, April 17, May 4, and July 26, 2007, requesting an amendment to the operating license for CNS to increase the storage capacity of its spent fuel pool (SFP) to maintain the capability to fully offload the core from

the reactor as the unit approaches the end of its operating license. To achieve this goal, the licensee plans to install two additional high-density spent fuel racks into the SFP. Existing spent fuel racks will remain in the pool in their current configuration. The proposed additional racks will have a closer assembly-to-assembly spacing to increase fuel storage capacity. The number of fuel assemblies that can be stored in the SFP would be increased from 2366 assemblies to 2651 assemblies (an increase of 285 assemblies).

The Need for the Proposed Action

An increase in spent fuel storage capacity is needed to maintain the capability for a full-core offload and to allow CNS to operate at full power until the next refueling outage. Loss of full-core offload capability occurred when the spent fuel was discharged to the SFP following Cycle 22 in January 2005. The licensee plans to install one of the additional high-density storage racks (with the capacity to store 117 fuel assemblies) immediately following issuance of the proposed amendment, with the second high-density storage rack (with the capacity to store 168 fuel assemblies) to be installed later if necessary, while keeping the existing racks in place. The additional capacity will ensure the capability of a full-core offload as the unit approaches the end of Cycle 25, at which point it will receive new fuel for Cycle 26 during the summer of 2009.

Environmental Impacts of the Proposed Action

The NRC has completed its safety evaluation of the proposed action and concludes that the proposed addition of two new storage racks to the SFP is acceptable. The details of the staff's safety evaluation will be provided in the license amendment that will be issued as part of the letter to the licensee approving the license amendment.

The staff has reviewed the licensee's plan for the expanded fuel storage capacity with respect to the radiological impact. The specifics of this review are presented below:

1. Radioactive Wastes

CNS uses waste treatment systems designed to collect and process gaseous, liquid, and solid waste that might contain radioactive material in a safe and controlled manner so that the discharges are in accordance with the regulatory standards of 10 CFR Part 20, and Appendix I to 10 CFR Part 50.

2. Solid Radioactive Wastes

The NRC staff reviewed the impact of the expanded fuel storage capacity on the production and release of radioactive waste during normal operations. The level of radioactive material in the pool water and the degree of water clarity determines the amount of solid waste produced by pool purification system resins. The licensee expects that during the fuel pool expansion work, small amounts of additional resins may be generated. This additional waste will be generated on a one-time basis. During normal operations, the licensee does not expect there to be a significant increase in the amount of solid radioactive wastes. Overall, the staff concludes that during routine operations, there will be no significant increase in the volume of solid radioactive wastes generated as a result of the proposed action.

3. Gaseous Radioactive Effluents

Radioactive gases that evolve from the surface of the pool water contribute to the plant's gaseous effluents. However, the levels of gaseous and particulate radioactivity in the pool water and in the area around the SFP are dominated by the most recent reactor offload to the SFP, not the older cooled fuel in the pool. Therefore, the storage of additional spent fuel assemblies resulting from the proposed action will have a minimal contribution to the gaseous effluents. The licensee has area radiation monitors in the immediate vicinity of the SFP, which monitor ambient airborne particulate and iodine radioactivity, and additional radiation monitors that monitor gaseous discharges into the environment. This radiation monitoring is performed to ensure continued compliance with the regulatory dose limits for the workers and members of the public. Overall, the staff concludes that during routine operations, there will be no significant increase in the amount of gaseous radiological effluents released into the area around the SFP and into the environment as a result of the proposed action.

4. Liquid Radiological Effluents

The number of stored spent fuel assemblies does not directly affect the release of radioactive liquids from the plant. The contribution from the stored fuel assemblies of radioactive materials in the SFP water is minor relative to other sources of activity, such as the reactor coolant system and its associated sub-systems. The volume of SFP water processed for discharge is independent of the quantity of stored spent fuel

assemblies. Therefore, the installation of the new fuel racks would not be expected to increase the amount of radioactive liquid wastes generated at the CNS. Overall, the staff concludes that during routine operations, there will be no significant increase in the amount of liquid radiological effluents released into the environment as a result of the proposed action.

5. Occupational Radiation Dose

During normal operations, personnel working in the fuel storage area are exposed to low levels of radiation from the SFP. Operating experience across the nuclear industry has shown that area dose rates originate primarily from radionuclides in the pool water, not the fuel itself, which is well shielded. The radiological conditions in the SFP area are typically dominated by the most recent discharge of spent fuel. The radioactivity inventory available for release into the general area from the older spent fuel, including the fuel from the expanded storage, is expected to be insignificant in comparison to freshly discharged fuel. During refueling and other fuel movement activities, pool water concentrations of radionuclides might be expected to increase to a small degree. However, the installation of the new fuel storage racks is not expected to cause any detectable increase in airborne activities or changes in the general area dose rates which might impact personnel working in the area.

All operations involved in the installation of the new fuel racks and the removal of any stored equipment or material from the SFP will be governed by plant procedures. The licensee's procedures incorporate the principle of keeping doses as low as reasonably achievable (ALARA), as required by NRC regulations.

The licensee does not expect to use underwater divers for the installation of the new fuel racks. However, in the event that diving operations are needed, the licensee is prepared to use specialized procedures and underwater radiation monitoring equipment to provide constant oversight and control to ensure the health and safety of the diver.

On the basis of our review of the CNS proposed expansion of the SFP storage capacity, the NRC staff concludes that the SFP work can be performed in a manner that will ensure that doses to the workers and the public, as well as the discharge of radioactive solid, gaseous, and liquid into the environment will be maintained within NRC regulations and standards. Therefore, there are no significant

radiological impacts associated with the proposed action.

6. Postulated Accident Considerations

The proposed modification increases the SFP storage capacity, but it does not change the method for handling spent fuel assemblies.

The proposed expansion of the SFP will not affect any of the assumptions or inputs used in evaluating the dose consequences of a fuel handling accident and, therefore, will not result in an increase in the doses from the previously analyzed postulated fuel handling accident. In summary, the staff has evaluated the proposed action and concludes that it does not increase the probability or consequences of a postulated accident.

7. Non-Radiological Impact

The proposed amendment to the current operating license of CNS does not modify land use at the site; no new facilities or laydown areas are needed to support the rerack or operation after rerack; therefore, the proposed amendment does not affect land use or land with historical or archeological sites.

With regard to potential non-radiological environmental impacts, the proposed action does not result in any significant changes to land use or water use, or result in any significant changes to the quality or quantity of effluents. The proposed action does not affect non-radiological plant effluents, and no changes to the National Pollution Discharge Elimination System permit are needed. No effects on the aquatic or terrestrial habitat in the vicinity or the plant, or to endangered or threatened species, or to the habitats of endangered or threatened species are expected.

The proposed action will not change the method of generating electricity or the method of handling any influents from the environment or non-radiological effluents to the environment. Therefore, no changes or different types of non-radiological environmental impacts are expected as a result of the proposed action.

8. Summary

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of radioactive effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational or public exposure. Accordingly, the staff concludes that there are no significant radiological environmental

impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action involves features located entirely within the restricted area, as defined in 10 CFR Part 20. It does not affect non-radiological plant effluents and is not expected to have any other environmental impact. Accordingly, the staff concludes that there are no significant non-radiological environmental impacts associated with the proposed action.

Environmental Impacts of Alternatives to the Proposed Action

1. Shipping Fuel to a Permanent Fuel Storage/Disposal Facility

Shipping of spent fuel to a high-level radioactive storage facility is an alternative to increasing onsite spent fuel storage capacity. The Department of Energy (DOE) has identified Yucca Mountain, Nevada, as the single candidate site for characterization as a potential geologic repository for high-level radioactive waste. However, this repository is not expected to begin receiving spent fuel until approximately 2025, provided that the DOE receives a license from the NRC. DOE plans to submit its license application for the proposed Yucca Mountain, Nevada, repository to the NRC in June 2008. Therefore, shipping spent fuel to the DOE repository is not considered an alternative to increased onsite spent fuel storage capacity at this time.

2. Shipping Fuel to a Reprocessing Facility

Reprocessing of spent fuel from CNS is not a viable alternative since there are no operating commercial reprocessing facilities in the United States. Therefore, spent fuel would have to be shipped to an overseas facility for reprocessing. However, this approach has never been used and would require approval by the Department of State as well as other entities. Additionally, the cost of spent fuel reprocessing is not offset by the current salvage value of the residual uranium; reprocessing would represent an added cost.

3. Shipping Fuel to Another Utility or SFP Site for Storage

The shipment of fuel to another utility's SFP for storage could provide short-term relief from the storage problem at CNS. The Nuclear Waste Policy Act of 1982 and 10 CFR Part 53, however, clearly place the responsibility for the interim storage of spent fuel with each owner or operator of a nuclear plant. SFPs at other nuclear stations

have been designed with the capacity to accommodate each of those units and, therefore, transferring spent fuel from CNS to these pools would eventually create fuel storage capacity problems at those stations. The shipment of fuel to another site is not an acceptable alternative because of increased fuel handling risks and additional occupational radiation exposure, as well as the fact that no additional storage capacity would be created.

4. Alternative Creation of Additional Storage Capacity

Alternative technologies that would create additional storage capacity include rod consolidation, new SFP construction, dry cask storage, and modular vault dry storage. Rod consolidation involves disassembling the spent fuel assemblies and storing the fuel rods from two or more assemblies in a stainless steel canister that can be stored in the spent fuel racks. Industry experience with rod consolidation is currently limited, primarily due to concerns for potential gap activity release due to rod breakage, the potential for increased fuel cladding corrosion due to some of the protective oxide layers being scraped off, and concern that the prolonged consolidation activity could interfere with ongoing plant operations.

Dry cask storage is a method of transferring spent fuel, after storage in the pool for several years, to high-capacity casks with passive-heat dissipation features. After loading, the casks are stored outdoors on a seismically qualified concrete pad. The casks provide housing for the spent fuel in shielded steel cylinders in a horizontal configuration within a reinforced concrete vault. The concrete vault provides missile and earthquake protection and radiation shielding. Though CNS is in the process of evaluating dry cask storage as a long-term storage option, it is not an alternative for resolving the current need for full-core offload capability due to the long lead time for an NRC license, time requirements for site preparation and construction, and the limited production of the dry casks used for storage. For these reasons, dry cask storage is not the licensee's preferred short-term method of storage.

5. Reduction of Spent Fuel Generation

Generally, improved usage of the fuel and/or operation at a reduced power level would be an alternative that would decrease the amount of fuel being stored in the pool and thus increase the amount of time before full-core offload capacity is lost. With extended burnup

of fuel assemblies, the fuel cycle would be extended and fewer offloads would be necessary. This is not an alternative for resolving the loss of full-core offload capacity that occurred as a result of the CNS refueling outage in January of 2005, because the spent fuel transferred to the pool for storage during this outage eliminated the licensee's ability to conduct a full-core offload. Operating the plant at a reduced power level would not make effective use of available resources, and would cause unnecessary economic hardship on the licensee and its customers. Therefore, reducing the amount of spent fuel generated by increasing burnup further or reducing power is not considered a practical alternative.

6. The No-Action Alternative

As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the amendment request would result in no change in current environmental impacts. The environmental impacts of the proposed amendment and this alternative are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Cooper Nuclear Station Final Environmental Impact Statement dated February 1973.

Agencies and Persons Contacted

In accordance with its stated policy, on August 27, 2007, the staff consulted with the Nebraska State official, Ms. J. Schmitt of the Nebraska Department of HHS Regulation and Licensure, Office of Radiological Health, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated October 17, 2006, as supplemented by letters dated February 7, April 17, May 4, and July 26, 2007. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike, Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide

Document Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site: <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or 301-415-4737, or send an e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 28th day of August, 2007.

For the Nuclear Regulatory Commission.

Carl F. Lyon,

*Project Manager, Plant Licensing Branch IV,
Division of Operating Reactor Licensing,
Office of Nuclear Reactor Regulation.*

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BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 70-143-CO; ASLBP No. 07-857-01-CO-BD01]

Nuclear Fuel Services, Inc.; Establishment of Atomic Safety and Licensing Board

Pursuant to delegation by the Commission dated December 29, 1972, published in the **Federal Register**, 37 FR 28,710 (1972), and the Commission's regulations, *see* 10 CFR 2.104, 2.300, 2.303, 2.309, 2.311, 2.318, and 2.321, notice is hereby given that an Atomic Safety and Licensing Board is being established to preside over the following proceeding:

Nuclear Fuel Services, Inc., Special
Nuclear Materials Facility
(Confirmatory Order)

This Board is being established in response to requests for hearing that were filed pursuant to a Notice of Publication of Confirmatory Order and Opportunity for Hearing (72 Fed. Reg. 41,528 (July 30, 2007)), regarding a Confirmatory Order issued to Nuclear Fuel Services, Inc. ("NFS") on February 21, 2007 that became immediately effective on the date of issuance. This proceeding arose from inspections and investigations at NSF by the NRC Staff that identified apparent violations for which escalated enforcement action was considered. The NRC Staff determined that its concerns regarding public health and safety could be resolved through confirmation of NFS's commitments as prescribed in the Confirmatory Order. Hearing requests have been submitted by: (1) Ken Silver, (2) R. Feher, (3) Linda Cataldo Modica on behalf of the Sierra

Club, (4) Wanda Sue Kelley, (5) Barbara A. O'Neal, and (6) A. Christine Tipton.

The Board is comprised of the following administrative judges:

Lawrence G. McDade, Chair, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

Dr. Richard F. Cole, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

Dr. Peter S. Lam, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

All correspondence, documents, and other materials shall be filed with the administrative judges in accordance with 10 CFR 2.302.

Issued at Rockville, Maryland, this 29th day of August 2007.

E. Roy Hawkens,

*Chief Administrative Judge, Atomic Safety
and Licensing Board Panel.*

[FR Doc. E7-17501 Filed 9-4-07; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-139]

Notice of License Termination for University of Washington Research Reactor (UWAR)

The U.S. Nuclear Regulatory Commission (NRC) is announcing the termination of facility Operating License No. R-73 for the University of Washington Research Reactor (UWAR).

The NRC has terminated the license of the decommissioned UWAR, at the University of Washington (UWA) in Seattle, Washington, and has released the site for unrestricted use. The UWAR was an Argonaut-type training and research reactor with an initial power output of 10 kilowatts, which later received authority to increase power output to 100 kilowatts. The reactor was permanently shut down on June 30, 1988. By application dated August 2, 1994, the licensee requested authorization to dismantle the UWAR and to dispose of the component parts, in accordance with the decommissioning plan submitted as part of the application. Opportunity for a hearing was afforded by "Notice of Proposed Issuance of Orders Authorizing Disposition of Component Parts and Terminating Facility License" published in the **Federal Register** on September 2, 1994 (59 FR 45738). No request for a hearing or petition for leave to intervene was filed following

notice of the proposed action. The NRC reviewed the application with respect to the Commission's rules and regulations and found that the dismantling and disposal of component parts as stated in the licensee's decommissioning plan are consistent with the regulations in 10 CFR Chapter I and are not inimical to the common defense and security or to the health and safety of the public. On May 1, 1995, the Commission issued the "Order Authorizing Dismantling of Facility and Disposition of Component Parts."

The licensee conducted remediation activities and completed final status surveys in October 2006. The licensee's request for termination of the license was supported by the submittal of a Final Status Survey Report (FSSR). The NRC completed its review of the UWAR FSSR submitted to NRC by letter dated December 13, 2006, as supplemented February 26 and March 12, 2007. The FSSR documented the level of residual radioactivity remaining at the facility and stated that compliance with the criteria in the NRC-approved decommissioning plan for the reactor has been demonstrated. The NRC staff verified that the criteria in the approved decommissioning plan had been met and determined that the facility and site met the criteria in 10 CFR 20.1402 for unrestricted use.

Pursuant to 10 CFR 50.82(b)(6), the NRC staff has concluded that the reactor has been decommissioned in accordance with the approved decommissioning plan and that the terminal radiation survey and associated documentation demonstrate that the facility and site may be released in accordance with the criteria in the NRC-approved decommissioning plan. Further, on the basis of the decommissioning activities carried out by UWA, the NRC's review of the licensee's FSSR, the results of NRC inspections conducted at the UWAR, and the results of NRC confirmatory surveys, the NRC has concluded that the decommissioning process is complete and the facility and site may be released for unrestricted use. Therefore Facility Operating License No. R-73 is terminated.

For further details with respect to the proposed action, see the licensee's letter dated December 13, 2006, as supplemented February 26 and March 12, 2007; and NRC Inspection Report No. 50-139/2006-204, dated May 21, 2007. The above referenced documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR) at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available