

currently residing in Range 63, Target Area 10, at Nellis Air Force Base. Therefore, no consultation is required under Section 106 of the National Historic Preservation Act.

On September 23, 2004, the staff consulted with two Nevada State officials, Mr. Stan Marshall of the Radiological Health Section of the Nevada State Health Division, Bureau of Health Protection Services and Ms. Jolene Johnson of the Nevada Division of Environmental Protection, regarding the environmental impact of the proposed action. Neither State Official had any comments regarding the draft EA. Additionally, the staff consulted with the Idaho State official, Mr. Doug Walker of the Idaho Department of Environmental Quality. On November 2, 2004, the State of Idaho, Department of Environmental Quality, provided comments regarding the draft EA, and those comments have been incorporated in the final EA.

III. Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

IV. Further Information

Documents related to this action, including the license amendment request and supporting documentation, are available electronically at the NRC's Electronic Reading Room at <http://www.nrc.gov/reading-rm/adams.html>. From this site, you may access the NRC's Agencywide Document Access and Management System (ADAMS), which provides text and image files of NRC's public documents. The ADAMS accession numbers for the documents related to this notice are: U.S. NRC Radioactive Materials License: Department of the Air Force, Docket Number 030-28641, License Number 42-23539-01AF; Request letter dated June 23, 2004, U.S. Department of the Air Force (ML041810555); NRC Technical Review of Code of Federal Regulations (10 CFR) Part 20.2002 request by U.S. Department of the Air Force (ML042120512); Safety Evaluation Report, August 5, 2005 (ML052170209); Environmental Assessment and FONSI, August 5, 2005 (ML052170216); Title 10 Code of Federal Regulations, 20.2002, "Method of Obtaining Approval of Proposed Disposal Procedures"; and Title 10 Code of Federal Regulations, Part 51, "Environmental Protection

Regulations for Domestic Licensing and Related Regulatory Functions."

If you do not have access to ADAMS or if there are problems with accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at (800) 397-4203, (301) 415-4737, or by e-mail to pdrc@nrc.gov. These documents may also be viewed electronically on the public computers located the NRC's PDR, O1 F21, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852. The PDR reproduction contractor will copy documents for a fee. The PDR is open from 7:45 a.m. to 4:15 p.m., Monday through Friday, except on Federal holidays.

Dated at Arlington, Texas, this 12th day of October 2005

For the Nuclear Regulatory Commission.

Jack E. Whitten,

Chief, Nuclear Materials Licensing Branch,
Division of Nuclear Materials Safety, Region IV.

[FR Doc. E5-5878 Filed 10-24-05; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-461]

Amergen Energy Company, LLC; Clinton Power Station, Unit 1; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an amendment to Facility Operating License No. NPF-62 issued to AmerGen Energy Company, LLC (AmerGen or the licensee), for operation of Clinton Power Station, Unit 1 (CPS), located in DeWitt County, Illinois. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would revise Technical Specification 4.3, "Fuel Assemblies," for CPS to reflect the increased fuel storage capacity in the spent fuel pool (SFP) and the addition of fuel storage capacity in the fuel cask storage pool. The proposed expansion will increase the total storage capacity from 2,512 to 4,159 fuel assemblies.

The proposed action is in accordance with the licensee's application dated August 18, 2004, as supplemented on May 13 and 25, June 14, and August 17, 2005.

The Need for the Proposed Action

The loss of full core discharge capability at CPS is projected to occur during the February 2006 refueling outage, based on current projections. To maintain spent fuel storage capability, AmerGen would like to expand SFP storage capacity. The proposed action would result in the increased fuel storage capacity in the SFP and the addition of fuel storage capacity in the fuel cask storage pool. The proposed expansion will increase the total storage capacity from 2,512 to 4,159 fuel assemblies. The additional capacity is expected to allow operation without loss of full-core discharge capability until the year 2016.

Environmental Impacts of the Proposed Action

Radioactive Waste Treatment

CPS uses waste treatment systems designed to collect and process gaseous, liquid, and solid waste that might contain radioactive material. These radioactive waste treatment systems were evaluated in the Final Environmental Statement (FES) for CPS, Unit 1, dated May 1982. The proposed changes to the SFP will not involve any change in the waste treatment systems described in the FES.

Gaseous Radioactive Wastes

The increase in the number of spent fuel assemblies stored in the SFP will potentially result in an increase in the radioactive gasses evolving from the pool. However, the level of gaseous radioactivity in the pool water is dominated by the most recent reactor core offload to the pool, not the fuel already stored in the pool. Therefore, the storage of additional aged spent fuel assemblies in the pool will have a minimal contribution to radioactivity in the pool. The overall release of radioactive gases from CPS will remain within the limits of Title 10, *Code of Federal Regulations* (10 CFR), Section 20.1301.

Solid Radioactive Wastes

Spent resins are generated by the processing of SFP water through the pools' purification system. These spent resins are disposed of as solid radioactive waste. Resin replacement is determined primarily by the requirement for water clarity and is normally done approximately once per year. No significant increase in the volume of solid radioactive waste is expected with the expanded storage capacity. During pool re-racking operations, small amounts of additional waste resin may be generated by the

pools' cleanup systems on a one-time basis. Additional solid radioactive waste will consist of the existing contaminated fuel storage racks. The old existing fuel storage racks will be washed down prior to being removed from the pool to remove as much contamination as possible. Then the racks will be shipped to a volume reduction facility for processing and subsequent disposal at a burial site. Shipping containers and procedures will conform to Federal regulations as specified in 10 CFR Part 71, "Packaging and Transportation of Radioactive Material," and to the requirements of any state through which the shipment may pass, as set forth by the state department of transportation.

Liquid Radioactive Wastes

The release of radioactive liquids will not be affected directly as a result of the SFP modifications. The SFP ion exchanger resins remove soluble radioactive materials from the pool water. When the resins are replaced, the small amount of resin sluice water that is released is processed by the radwaste systems. As previously stated, the frequency of resin replacement may increase slightly during the installation of the new racks. However, the increase in the amount of radioactive liquid released to the environment as a result of the proposed SFP expansion is expected to be negligible.

Occupational Dose Consideration

All operations involved in the fuel rack installations will follow detailed procedures prepared in accordance with as low as reasonably achievable (ALARA) principles. Personnel performing the re-racking operation will be given pre-job briefings to ensure awareness of job responsibilities and necessary precautions. Radiation protection personnel at CPS will monitor and control work, personnel traffic, and equipment movement in the SFP area to minimize contamination and assure that exposures are maintained ALARA. Personnel monitoring equipment (including thermoluminescence dosimeter (TLDs)), protective clothing, and respiratory protective equipment will be issued as required. Alarming dosimeters will be used as needed to confirm exposure and dose rates to workers.

The licensee plans to use divers in the pool to remove underwater interferences and assist in fuel storage rack removal. Procedures for controlling diving operations will comply with the guidance in Regulatory Guide 8.38, "Control of Access to High and Very High Radiation Areas in Nuclear Power Plants." During the diving operations,

the licensee estimates that dose rates will average from 20 to 40 mrem/hr. Special precautions such as physical barriers or tethers will be used to prevent a diver from coming in close proximity to highly radioactive materials in the pool. The diver will be confined to a safe diving area within the pool, which will be clearly delineated in the pre-job brief as well as physically marked in the pool. The diver will be visually monitored, either directly or remotely, at all times during the dive. In addition, the diver will be monitored by a remote dose telemetry system. This system enables the radiation protection personnel supervising the dive to obtain the dose being delivered to the diver's body. The diver will have a hand-held probe to complete radiological surveys when entering the water. Divers exiting the pool will be monitored for radiation and contamination, as will all items removed from the pool. Appropriate measures will be taken to minimize the spread of contamination. The existing fuel racks that are removed from the pool will be rinsed and surveyed as they break the water's surface, allowed to "drip dry," and then placed in plastic shipping bags to contain any contamination until they are placed in shipping containers to be taken offsite for disposal.

The increased storage capacity will not affect dose rates in areas adjacent to the SFP and transfer canal. The concrete side walls of the SFP provide sufficient shielding that the maximum dose rate in adjacent areas from fuel in the SFP is calculated to be 2 mrem/hr, if the pool is completely filled with freshly offloaded fuel. The walls of the fuel cask storage pool are not as thick, and the licensee's shielding calculations indicate that filling the racks that are proposed to be installed in the fuel cask storage pool with freshly offloaded fuel could result in dose rates of up to 26 mrem/hr in adjacent areas. This could be mitigated by filling the outer (peripheral) three rows of the storage cells with older (more decayed) fuel, thus reducing the maximum dose rate in the adjacent areas to 4.4 mrem/hr. The licensee will implement administrative controls to ensure that fuel stored in the peripheral storage cells will have been stored outside of the reactor for a minimum of 10 years, allowing sufficient decay time.

On the basis of its review of the licensee's proposal, the NRC staff concludes that the CPS SFP re-racking operations can be performed in a manner that will ensure that doses to workers will be maintained ALARA and that the generation of additional solid radioactive waste will be minimized.

The staff concludes that the projected dose for the project of 7 to 14 person-rem is in the range of doses for similar modifications at other nuclear plants.

Accident Considerations

The licensee evaluated the impact of newly installed higher density storage racks in the SFP and fuel storage in the fuel cask storage pool on the current design basis accident (DBA) dose analyses, as discussed in the CPS Updated Safety Analysis Report. The DBAs that are potentially affected by the proposed change to the SFP storage capacity are the fuel handling accident (FHA) and the cask drop accident. By Amendment No. 147, dated April 3, 2002, the CPS licensing basis for the FHA was changed by a selective implementation of an alternative source term, per the provisions of 10 CFR 50.67. In support of that amendment request, AmerGen demonstrated that the radiological consequences of an FHA, either in the containment or in the fuel building, are within the offsite and control room dose acceptance criteria specified in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants," and General Design Criterion 19 of 10 CFR Part 50, Appendix A, and well within the dose criteria given in 10 CFR 50.67.

The NRC staff performed a review of the licensee's analysis of the proposed action on DBA dose analyses. Adding additional spent fuel storage does not increase the amount of fuel assumed to be damaged in an FHA, and the proposed action does not significantly change the source term in the DBA; therefore, the staff finds that the current licensing basis FHA dose analysis remains applicable after the expansion of the spent fuel storage capacity. The licensee plans to install spent fuel storage racks in the fuel cask storage pool. The licensee will implement administrative controls to ensure that fuel will be removed from the racks in the fuel cask storage pool prior to any fuel cask being moved in the area. Therefore, there would be no damage to spent fuel or radiological consequences as a result of a cask drop on the empty fuel storage racks in the fuel cask storage area. Based on its review, the staff finds that the current licensing basis analysis of the cask drop accident remains bounding with respect to radiological consequences.

During removal and installation of fuel storage racks in the SFP and fuel cask storage pool, AmerGen will ensure that all work will be controlled and performed in strict accordance with specific written guidance. Any

movement of fuel assemblies required to support removal and installation of racks will be performed as during normal refueling operations, and no shipping cask movement will be performed during this time frame. The licensee will determine and follow safe load paths and written procedures to ensure that no racks are carried over any portions of the existing fuel storage racks containing fuel assemblies.

Based on its review, the staff concludes that the current DBA dose analyses remain bounding for the installation of expanded spent fuel storage capacity in the SFP and fuel cask storage pool.

The proposed action will not significantly increase the probability or consequences of accidents. No changes are being made in the types of effluents that may be released off site. There is no significant increase in the amount of any effluent released off site. There is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not have a potential to affect any historic sites. It does not affect non-radiological plant effluents and has no other environmental impact. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Shipping Fuel to a Permanent Federal Fuel Storage/Disposal Facility

Shipment of spent fuel to a high-level radioactive storage facility is an alternative to increasing the onsite spent fuel storage capacity. However, the U.S. Department of Energy's (DOE's) proposed high-level radioactive waste repository is not expected to begin receiving spent fuel in the near future. Therefore, shipping the spent fuel to the DOE repository is not considered an alternative to increased onsite fuel storage capacity at this time.

Shipping Fuel to a Reprocessing Facility

Reprocessing of spent fuel from CPS is not a viable alternative since there are no operating commercial reprocessing facilities in the United States. Therefore, spent fuel would have to be shipped to an overseas facility for reprocessing. However, this approach has never been used and it would require approval by

the Department of State as well as other entities.

Additionally, the cost of spent fuel reprocessing is not offset by the salvage value of the residual uranium; reprocessing represents an added cost.

Shipping the Fuel Offsite to Another Utility or another Exelon/AmerGen Site

The shipment of fuel to another utility or transferring fuel to another of the licensee's facilities would provide short-term relief from the shortage of SFP storage at CPS. However, the Nuclear Waste Policy Act of 1982, Subtitle B, Section 131(a)(1) clearly places the responsibility for the interim storage of spent fuel with each owner or operator of a nuclear plant. The SFPs at the other reactor sites were designed with capacity to accommodate spent fuel from those particular sites. Therefore, transferring spent fuel from CPS to other sites would create storage capacity problems at those locations. The shipment of spent fuel to another site or transferring it to another Exelon/AmerGen site is not an acceptable alternative because of increased fuel handling risks and additional occupational radiation exposure, as well as the fact that no additional storage capacity would be created.

Alternatives Creating Additional Storage Capacity

Alternative technologies that would create additional storage capacity include rod consolidation, dry cask storage, modular vault dry storage, and constructing a new pool. Rod consolidation involves disassembling the spent fuel assemblies and storing the fuel rods from two or more assemblies into a stainless steel canister that can be stored in the spent fuel racks. Industry experience with rod consolidation is currently limited, primarily due to concerns for potential gap activity release due to rod breakage, the potential for increased fuel cladding corrosion due to some of the protective oxide layer being scraped off, and because the time-consuming consolidation activity could interfere with ongoing plant operations. Dry cask storage is a method of transferring spent fuel, after storage in the pool for several years, to high capacity casks with passive heat dissipation features. After loading, the casks are stored outdoors on a seismically qualified concrete pad. Concerns for dry cask storage include the need for special security provisions and high cost. Vault storage consists of storing spent fuel in shielded stainless steel cylinders in a horizontal configuration in a reinforced concrete vault. The concrete vault provides

missile and earthquake protection and radiation shielding. Concerns for vault dry storage include security, land consumption, eventual decommissioning of the new vault, the potential for fuel or clad rupture due to high temperatures, and high cost. The alternative of constructing and licensing new SFPs is not practical for CPS because such an effort would require about 10 years to complete and would be an expensive alternative.

The alternative technologies that could create additional storage capacity involve additional fuel handling with an attendant opportunity for an FHA, involve higher cumulative dose to workers affecting the fuel transfers, require additional security measures that are significantly more expensive, and would not result in a significant reduction in environmental impacts compared to the proposed re-racking modifications.

Reduction of Spent Fuel Generation

Generally, improved usage of the fuel and/or operation at a reduced power level would be an alternative that would decrease the amount of fuel being stored in the SFPs and, thus, increase the amount of time before the maximum storage capacities of the SFPs are reached. However, operating the plant at a reduced power level would not make effective use of available resources, and would cause unnecessary economic hardship on the licensee and its customers. Therefore, reducing the amount of spent fuel generated by reducing power is not considered a practical alternative.

Impact on SFP Storage From Increasing Length of Fuel Cycle

By letter dated May 20, 2004, as supplemented May 23 and September 30, 2005, the licensee requested changes to the Technical Specification Surveillance Requirement frequencies to support 24-month fuel cycles at CPS in accordance with the guidance of Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle." Currently, this request is under review by the NRC staff. If this request is approved, CPS will experience a loss of full core discharge capability sooner. Therefore, this is not a practical alternative to the proposed action.

The No-Action Alternative

The NRC staff also considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The

environmental impacts of the proposed action and this alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resources than those previously considered in the FES for CPS, Unit 1, dated May 1982.

Agencies and Persons Contacted

In accordance with its stated policy, on September 27, 2005, the NRC staff consulted with Illinois State Official, Frank Niziolek of the Illinois Emergency Management Agency, regarding the environmental impact of the proposed action. The state official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated August 18, 2004, as supplemented by letters dated May 13 and 25, June 14, and August 17, 2005. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff at 1-800-397-4209, or 301-415-4737, or send an e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 17th day of October, 2005.

For the Nuclear Regulatory Commission.

Kahtan N. Jabbar,

Sr. Manager, Section 2, Project Directorate III, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. E5-5874 Filed 10-24-05; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Sunshine Act Meeting

AGENCY HOLDING THE MEETINGS: Nuclear Regulatory Commission.

DATE: Weeks of October 24, 31, November 7, 14, 21, 28, 2005.

PLACE: Commissioner's Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and closed.

MATTERS TO BE CONSIDERED:

Week of October 24, 2005

Wednesday, October 26, 2005

1:20 p.m.

Affirmation Session (Public Meeting) (Tentative).

- a. U.S. Army (Jefferson Proving Ground Site) (Materials License Amendment) (tentative).
- b. Dominion Nuclear Connecticut, Inc. (Millstone Nuclear Power Station, Units 2 and 3), Docket Nos. 50-336-LR & 50-423-LR, LBP-05-16 (July 20, 2005) (tentative).
- c. Amergen Energy Co. (Three Mile Island Nuclear Station, Unit 1), Docket No. 50-289-LT-2 (tentative).
- d. Exelon Generation Company, LLC & PSEG Nuclear, LLC (Peach Bottom Atomic Power Station, Units 2 and 3, Docket Nos. 50-277-LT & 50-278-LT (tentative).

1:30 p.m.

Discussion of Security Issues (Closed—Ex. 1).

Thursday, October 27, 2005

10 a.m.

Discussion of Security Issues (Closed—Ex. 1).

Week of October 31, 2005—Tentative

Tuesday, November 1, 2005

9:30 a.m.

Briefing on Implementation of Davis-Besse Lessons Learned Task Force (DBLLTF) Recommendations (Public Meeting). (Contact: Brendan Moroney, (301) 415-3974).

This meeting will be Webcast live at the Web address—<http://www.nrc.gov>.

Week of November 7, 2005—Tentative

There are no meetings scheduled for the Week of November 7, 2005.

Week of November 14, 2005—Tentative

There are no meetings scheduled for the Week of November 14, 2005.

Week of November 21, 2005—Tentative

Monday, November 21, 2005

9:30 a.m.

Briefing on Status of New Reactor Issues, Part 1 (Public Meeting). (Contact: Laura Dudes, (301) 415-0146).

1:30 p.m.

Briefing on Status of New Reactor Issues, Part 2 (Public Meeting). (Contact: Laura Dudes, (301) 415-0146).

This meeting will be Webcast live at the Web address—<http://www.nrc.gov>.

Week of November 28, 2005—Tentative

Wednesday, November 30, 2005

9:30 a.m.

Briefing on EEO Program (Public Meeting). (Contact: Corentis Kelley, (301) 415-7380).

This meeting will be Webcast live at the Web address—<http://www.nrc.gov>.

*The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (recording)—(301) 415-1292. Contact person for more information. Michelle Schroll, (301) 415-1662.

Additional Information

By a vote of 5-0 on October 18, 2005, the Commission determined pursuant to U.S.C. 552b(e) and § 9.107(a) of the Commission's rules that "Affirmation of a. Louisiana Energy Services, L.P. (National Enrichment Facility), Intervenor's Petition for Review of LBP-05-13 (Decision on Environmental Contentions); b. Louisiana Energy Services, L.P. (National Enrichment Facility), Licensing Board's Referral of Memorandum and Order Rejecting Amended and Supplemental Contentions; and c. Private Fuel Storage (Independent Spent Fuel Storage Installations) Docket No. 72-22-ISFSI; Review of Board's September 15, 2005 order regarding safeguards redactions" be held October 19, 2005, and on less than one week's notice to the public.

The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/what-we-do/policy-making/schedule.html>.

The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (e.g., braille, large print), please notify the NRC's Disability Program Coordinator, August Spector, at (301) 415-7080, TDD: (301) 415-2100, or by e-mail at aks@nrc.gov. Determinations on requests for reasonable accommodation will be made on a case-by-case basis.