

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–387 and 50–388]

PPL Susquehanna, LLC, Allegheny Electric Cooperative, Inc., Susquehanna Steam Electric Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from Title 10 of the Code of Federal Regulations (10 CFR), part 50, section 50.60(a), and Appendix G, for Facility Operating License Nos. NPF–14 and NPF–22 issued to PPL Susquehanna, LLC (PPL, the licensee), for operation of the Susquehanna Steam Electric Station (SSES), Units 1 and 2 (SSES–1 and 2), located in Luzerne County, Pennsylvania. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would allow PPL to use American Society of Mechanical Engineers (ASME) Code Case N–640 as the basis for establishing the fracture toughness values used in pressure-temperature (P–T) limit calculations. Section 50.60(a) of 10 CFR part 50 requires nuclear power reactors to meet the fracture toughness requirements set forth in 10 CFR part 50, Appendix G. Appendix G of 10 CFR part 50 requires that P–T limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak rate testing conditions. Specifically, 10 CFR part 50, Appendix G, states, “The appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions.” Appendix G of 10 CFR part 50 specifies that the requirements for these limits are the ASME Boiler and Pressure Vessel Code (ASME Code), Section XI, Appendix G, limits. Code Case N–640 permits application of the lower bound static initiation fracture toughness value equation (K_{Ic} equation) as the basis for establishing the P–T curves in lieu of using the lower bound crack arrest fracture toughness value equation (i.e., the K_{Ia} equation, the method invoked by Appendix G to Section XI of the ASME Code) as the basis for the curves.

The proposed action is in accordance with the licensee’s application for exemption dated July 17, 2001, as

supplemented by letters dated July 26, and October 15, 2001.

The Need for the Proposed Action

ASME Code Case N–640 is needed to revise the method used to determine the P–T limits, since continued use of the present curves unnecessarily restricts the reactor coolant system (RCS) P–T operating window. The RCS P–T operating window is defined by the RPV P–T operating and test limit curves developed in accordance with the ASME Code, Section XI, Appendix G. Continued operation of SSES–1 and 2, with these P–T curves without the relief provided by ASME Code Case N–640 would unnecessarily require the licensee to maintain the RCS temperature in a limited, high-temperature (over 200 °F) operating band during the pressure test. This results in challenges to plant operators in maintaining the RCS within the narrow allowable temperature band and challenges to personnel safety due to the high ambient drywell temperatures. Implementation of the proposed P–T curves, as allowed by ASME Code Case N–640, does not significantly reduce the margin of safety and would eliminate the challenges to plant operators and personnel safety by allowing the pressure test to be conducted at a lower coolant temperature.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the exemption described above would provide an adequate margin of safety against brittle failure of the SSES–1 and 2 RPVs.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not have a potential to affect any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the “no-action” alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resource than those previously considered in the Final Environmental Statement for the Susquehanna Steam Electric Station, dated June 1981.

Agencies and Persons Consulted

On December 17, 2001, the staff consulted with the Pennsylvania State official, Mr. Michael Murphy of the Pennsylvania Department of Environmental Protection, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee’s letter dated July 17, 2001, as supplemented by letters dated July 26, and October 15, 2001. Documents may be examined, and/or copied for a fee, at the NRC’s Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems Public Library (ADAMS) component on the NRC Web site, <http://www.nrc.gov> (the Public Electronic Reading Room). Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1–800–397–4209, or 301–415–4737, or by e-mail at pdr@nrc.gov.

Dated at Rockville, Maryland, this 30th day of January 2002.

For the Nuclear Regulatory Commission.
Joel T. Munday,
*Acting Chief, Section 1, Project Directorate
 I, Division of Licensing Project Management,
 Office of Nuclear Reactor Regulation.*
 [FR Doc. 02-2738 Filed 2-4-02; 8:45 am]
BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Sunshine Act Meeting

AGENCY HOLDING THE MEETING: Nuclear Regulatory Commission.

DATE: Weeks of February 4, 11, 18, 25, March 4, 11, 2002.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and Closed.

MATTERS TO BE CONSIDERED:

Week of February 4, 2002

Wednesday, February 6, 2002

9:25 a.m.

Affirmation Session (Public Meeting) (If needed)

9:30 a.m.

Briefing on Equal Employment Opportunity (EEO) Program (Public Meeting) (Contact: Irene Little, 301-415-7380)

Week of February 11, 2002—Tentative

There are no meetings scheduled for the Week of February 11, 2002.

Week of February 18, 2002—Tentative

Tuesday, February 19, 2002

1:55 p.m.

Affirmation Session (Public Meeting) (If needed)

2 p.m.

Meeting with the Advisory Committee on the Medical Uses of Isotopes (ACMUI) (Public Meeting) (Contact: Angela Williamson, 301-415-5030)

This meeting will be webcast live at the Web address—www.nrc.gov

Week of February 25, 2002—Tentative

Friday, March 1, 2002

9:30 a.m.

Briefing on Status of Office of the Chief Financial Officer (OCFO) Programs, Performance, and Plans (Public Meeting) (Contact: Lars Solander, 301-415-6080)

This meeting will be webcast live at the Web address—www.nrc.gov

Week of March 4, 2002—Tentative

Monday, March 4, 2002

2 p.m.

Briefing on Status of Nuclear Waste

Safety (Public Meeting) (Contact: Claudia Seelig, 301-415-7243)

This meeting will be webcast live at the Web address—www.nrc.gov

Week of March 11, 2002—Tentative

There are no meetings scheduled for the Week of March 11, 2002.

* The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (recording)—(301-415-1292. Contact person for more information: David Louis Gamberoni (301-415-1651).

Additional Information

By a vote of 5-0 on January 29 and 30, the Commission determined pursuant to U.S.C. 552b(e) and § 9.107(a) of the Commission's rules that "Affirmation of 1) Dominion Nuclear Connecticut Inc. (Millstone Nuclear Power Station, Units 2 and 3) Petition for Reconsideration of CLI-01-24 and 2) Duke Cogema Stone & Webster (Savannah River Mixed Oxide Fuel Fabrication Facility); Georginas Against Nuclear Energy's Motion for Reconsideration of CLI-01-28" be held on January 30, and on less than one week's notice to the public.

The NRC Commission Meeting Schedule can be found on the Internet at: www.nrc.gov

This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to the distribution, please contact the Office of the Secretary, Washington, D.C. 20555 (301-415-1969). In addition, distribution of this meeting notice over the Internet system is available. If you are interested in receiving this Commission meeting schedule electronically, please send an electronic message to dkw@nrc.gov.

Dated: January 31, 2002.

David Louis Gamberoni,
Technical Coordinator, Office of the Secretary.

[FR Doc. 02-2801 Filed 2-1-02; 10:23 am]

BILLING CODE 7590-01-M

NUCLEAR REGULATORY COMMISSION

Biweekly Notice; Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations

I. Background

Pursuant to Public Law 97-415, the U.S. Nuclear Regulatory Commission (the Commission or NRC staff) is publishing this regular biweekly notice. Public Law 97-415 revised section 189 of the Atomic Energy Act of 1954, as

amended (the Act), to require the Commission to publish notice of any amendments issued, or proposed to be issued, under a new provision of section 189 of the Act. This provision grants the Commission the authority to issue and make immediately effective any amendment to an operating license upon a determination by the Commission that such amendment involves no significant hazards consideration, notwithstanding the pendency before the Commission of a request for a hearing from any person.

This biweekly notice includes all notices of amendments issued, or proposed to be issued from January 11, 2002 through January 24, 2002. The last biweekly notice was published on January 22, 2002 (67 FR 2917).

Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The Commission has made a proposed determination that the following amendment requests involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The basis for this proposed determination for each amendment request is shown below.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received before action is taken. Should the Commission take this action, it will publish in the