

and Health Administration, 4015 Wilson Boulevard, Room 627, Arlington, Virginia 22203. All comments must be postmarked or received in that office on or before June 11, 1999. Copies of these petitions are available for inspection at that address.

Dated: May 4, 1999

**Carol J. Jones,**

*Acting Director, Office of Standards, Regulations, and Variances.*

[FR Doc. 99-12028 Filed 5-11-99; 8:45 am]

BILLING CODE 4510-43-P

## NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice 99-066]

### NASA Advisory Committees; Renewal of NASA's Advisory Committee Charters

**AGENCY:** National Aeronautics and Space Administration (NASA).

**ACTION:** Notice of Renewal of the Charters of NASA's Advisory Committees.

**SUMMARY:** Pursuant to section 14(b)(1) of the Federal Advisory Committee Act, Pub. L. 92-463, and after consultation with the Committee Management Secretariat, General Services Administration, the Administrator of the National Aeronautics and Space Administration has determined that a renewal of NASA's nine advisory committees is in the public interest in connection with the performance of duties imposed upon NASA by law. The structure and duties of these committees are unchanged. However, the Aeronautics and Space Transportation Technology Advisory Committee has been renamed to be the Aero-Space Technology Advisory Committee and some administrative language has been changed in each of the charters. The charter filing date is April 29, 1999 for each of the nine charters.

NASA's nine advisory committees are:

- NASA Aerospace Safety Advisory Panel
- NASA Advisory Council (NAC)
- NAC Technology and Commercialization Advisory Committee
- NAC Minority Business Resource Advisory Committee
- NAC Advisory Committee on the International Space Station
- NAC Aero-Space Technology Advisory Committee
- NAC Space Science Advisory Committee
- NAC Life and Microgravity Sciences and Applications Advisory Committee

NAC Earth System Science and Applications Advisory Committee

**FOR FURTHER INFORMATION CONTACT:** Ms. Kathy Dakon, Assistant Advisory Committee Management Officer, Mail Code Z, National Aeronautics and Space Administration, Washington, DC 20546 (202) 358-0732.

**SUPPLEMENTARY INFORMATION:** NASA Advisory Council and its committees information is available on the world wide web at: <http://www.hq.nasa.gov/office/codeq/codeq-1.htm> and <http://www.hq.nasa.gov/office/codez/nac.htm>.

Dated: May 4, 1999.

**Matthew M. Crouch,**

*Advisory Committee Management Officer, National Aeronautics and Space Administration.*

[FR Doc. 99-11895 Filed 5-11-99; 8:45 am]

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## NEIGHBORHOOD REINVESTMENT CORPORATION

### Sunshine Act Meeting

Twenty-First Annual Meeting of the Board of Directors

**TIME & DATE:** 2:00 p.m., Wednesday, May 24, 1999.

**PLACE:** Neighborhood Reinvestment Corporation, 1325 G Street, NW, Suite 800, Board Room, Washington, DC 20005.

**STATUS:** Open.

**CONTACT PERSON FOR MORE INFORMATION:** Jeffrey T. Bryson, General Counsel/Secretary (202) 220-2372.

**AGENDA:**

- I. Call to Order
- II. Approval of Minutes: March 3, 1999, Regular Meeting
- III. Resolution of Appreciation
- IV. Election of Chairman
- V. Election of Vice Chairman
- VI. Committee Appointments:
  - a. Audit Committee
  - b. Budget Committee
  - c. Personnel Committee
  - d. Homeownership Oversight Special Committee
- VII. Election of Officers
- VIII. Board Appointments:
  - a. Internal Audit Director
  - b. Assistant Secretary/Paralegal
- IX. Audit Committee Report: May 11, 1999
- X. Treasurer's Report
- XI. Executive Director's Quarterly Management Report
- XII. Adjourn

**Jeffrey T. Bryson,**

*General Counsel/Secretary.*

[FR Doc. 99-12166 Filed 5-10-99; 3:42 pm]

BILLING CODE 7570-01-M

## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-498 and 50-499]

### STP Nuclear Operating Company (South Texas Project Electric Generating Station, Units 1 and 2); Exemption

#### I

STP Nuclear Operating Company is the holder of Facility Operating License No. NPF-76 and Facility Operating License No. NPF-80, which authorizes operation of the South Texas Project (STP), Units 1 and 2. The licenses provide, among other things, that the licensee is subject to all rules, regulations, and orders of the Commission now or hereafter in effect.

These facilities consist of two pressurized-water reactors at the licensee's site located in Matagorda County, Texas.

#### II

Section 50.60(a) to Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50 requires, in part, that except as provided in Section 50.60(b), all light-water nuclear power reactors, other than reactor facilities for which the certifications required under Section 50.82(a)(1) have been submitted, must meet the fracture toughness requirements for the reactor coolant pressure boundary set forth in Appendix G of 10 CFR Part 50. Section 50.60(b) of 10 CFR Part 50 states that proposed alternatives to the described requirements of Appendix G of Part 50 or portions thereof may be used when an exemption is granted by the Commission under 10 CFR 50.12.

#### III

By letter dated March 18, 1999, STP Nuclear Operating Company requested that the NRC exempt STP, Units 1 and 2, from the application of specific requirements of 10 CFR 50.60 and Appendix G to 10 CFR 50. Specifically, STP Nuclear Operating Company proposes to use American Society of Mechanical Engineers (ASME) Code Case N-514 to permit setting the pressure setpoint of STP's cold overpressure mitigation system (COMS) such that the pressure-temperature (P-T) limits required by Appendix G of 10 CFR Part 50 could be exceeded by 10 percent during a low temperature pressure transient.

The Commission has established requirements in 10 CFR Part 50 to protect the integrity of the reactor coolant system pressure boundary. As a part of these, Appendix G of 10 CFR

Part 50 requires that P-T limits be established for reactor pressure vessels during normal operation and vessel hydrostatic testing. As stated in Appendix G, "The appropriate requirements on . . . the pressure-temperature limits . . . must be met for all conditions." In order to avoid approaching these P-T limit curves and provide pressure relief during low temperature overpressurization (LTOP) events, pressurized water reactor licensees have installed protection systems (COMS/LTOP) as part of the reactor coolant system (RCS) pressure boundary. STP Nuclear Operating Company is required, as part of the STP Technical Specifications, to develop, update, and submit reactor vessel P-T limits and COMS setpoints for NRC review and approval.

STP Nuclear Operating Company determined that the exemption request from the provisions of 10 CFR 50.60 and Appendix G was necessary since these regulations require, as previously noted, that reactor vessel conditions not exceed the P-T limits established by Appendix G. In referring to 10 CFR 50.12 on specific exemptions, STP Nuclear Operating Company cited special circumstances regarding achievement of the underlying purpose of the regulation as its basis for requesting this exemption [10 CFR 50.12(a)(2)(ii)].

STP Nuclear Operating Company noted in support of the 10 CFR 50.12(a)(2)(ii) criteria that the underlying purpose of the subject regulation is to establish limits to protect the reactor vessel from brittle failure during low temperature operation and that the COMS provides a physical means of assuring that operation remains within these limits. STP Nuclear Operating Company proposed that establishing the COMS pressure setpoint in accordance with the N-514 provisions, such that the vessel pressure would not exceed 110 percent of the P-T limit allowables, would still provide an acceptable level of safety and mitigate the potential for an inadvertent actuation of the COMS. The use of N-514 was based on the conservatism that have been explicitly incorporated into the procedure for developing the P-T limit curves. This procedure, referenced from Appendix G to Section XI of the ASME Code, includes the following conservatisms: (1) a safety factor of 2 on the pressure stresses; (2) a margin factor applied to  $RT_{NDT}$  using Regulatory Guide 1.99, Revision 2, "Radiation Embrittlement of Reactor Vessel Materials"; (3) an assumed  $\frac{1}{4}$  thickness flaw with a 6:1 aspect ratio; and (4) a limiting material toughness based on dynamic and crack arrest data.

In addition, STP Nuclear Operating Company stated that a COMS pressure setpoint must be sufficiently high to prevent the inadvertent actuation of the COMS as a result of normal operating pressure surges. STP Nuclear Operating Company requests use of Code Case N-514 to incorporate pressure instrumentation uncertainty in P-T limit calculations, while providing an operating band that permits system makeup and pressure control. Such an inadvertent actuation could lead to the unnecessary release of reactor coolant inside containment and could introduce undesirable thermal transients in the RCS.

The Commission has determined that the application of 10 CFR 50.60 in these particular circumstances is not necessary to achieve the underlying purpose of that rule and that the use of Code Case N-514 would meet the underlying intent of the regulation. Based upon a consideration of the conservatisms, which are explicitly defined in the Appendix G methodology, it was concluded that permitting the COMS setpoint to be established such that the vessel pressure would not exceed 110 percent of the limit defined by the P-T limit curves would provide an adequate margin of safety against brittle failure of the reactor vessel. This is also consistent with the determination that has been reached for other licensees under similar conditions based on the same conditions. Therefore, the exemption requested under the special circumstances of 10 CFR 50.12(a)(2)(ii) was found to be acceptable. The staff also agrees that limiting the potential for inadvertent COMS actuation may improve plant safety.

#### IV

The Commission has determined that, pursuant to 10 CFR 50.12, this exemption is authorized by law, will not present an undue risk to the public health and safety, is consistent with the common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants STP Nuclear Operating Company an exemption from the requirements of 10 CFR 50.60 in order to apply ASME Code Case N-514 for determining STP's cold overpressurization mitigation system pressure setpoint.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant effect on the quality of the human environment (64 FR 23689).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 4th day of May 1999.

For the Nuclear Regulatory Commission.

**John A. Zwolinski,**

*Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.*

[FR Doc. 99-11997 Filed 5-11-99; 8:45 am]

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-445 and 50-446]

### Texas Utilities Electric Company (Comanche Peak Steam Electric Station, Units 1 and 2); Exemption

#### I.

Texas Utilities Electric Company (the licensee/TU Electric) is the holder of Facility Operating Licenses No. NPF-87 and No. NPF-89, which authorize operation of the Comanche Peak Steam Electric Station (CPSES), Units 1 and 2. The licenses provide, among other things, that the licensee is subject to all rules, regulations, and orders of the Commission now or hereafter in effect.

These facilities consist of two pressurized-water reactors at the licensee's site located in Somervell County, Texas.

TU Electric seeks this exemption to the 2 percent above licensed power level assumption to allow for uncertainties specified by Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, Appendix K, "ECCS [Emergency Core Cooling System] Evaluation Models," Section I.A., to support license amendments for modest increases of up to 1 percent in the licensed power levels for both units. This will result in an exemption from the requirements of 10 CFR Part 50, Appendix K to allow ECCS evaluation model assumptions to be conducted at no less than 1.01 times licensed power level. The licensee seeks this exemption based on its proposed use of a new feedwater flow measurement system to allow more accurate measurement of thermal power (known as the Leading Edge Flowmeter (LEFM) System), manufactured by Caldon, Inc. The LEFM is described in Caldon, Inc., Topical Report ER-80P, "Improving Thermal Power Accuracy and Plant Safety While Increasing Operating Power Level Using the LEFM System." The subject topical report was approved subject to the limitations stated in a letter and Safety Evaluation (SE) dated March 8, 1999.