

to prevent brittle fracture of the reactor pressure vessel from occurring during low temperature operation.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that the use of Code Case N-514 as an alternative method for establishing the setpoints for the LTOP systems at DCPD Units 1 and 2 would provide an adequate margin of safety against brittle fracture of the DCPD reactor vessels.

The proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not involve any historic sites. It does not affect non-radiological plant effluents and has no other environmental impact. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the Commission concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for Diablo Canyon Power Plant dated May 1973, and the Addendum dated May 1976.

Agencies and Persons Consulted

In accordance with its stated policy, on April 22, 1999, the staff consulted with the California State official, Mr. Steve Hsu of the Radiologic Health Branch of the State Department of Health Services, regarding the environmental impact of the proposed amendments. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the Commission concludes that the proposed amendments will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's application dated September 3, 1998, as supplemented dated January 22, February 5, and March 17, 1999, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the California Polytechnic State University, Robert E. Kennedy Library, Government Documents and Maps Department, San Luis Obispo, California 93407.

Dated at Rockville, Maryland, this 28th day of April 1999.

For the Nuclear Regulatory Commission.

Steven D. Bloom,

Project Manager, Section 2, Project Directorate IV & Decommissioning, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-11117 Filed 4-30-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-498 and 50-499]

STP Nuclear Operating Co., South Texas Project, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from Facility Operating License No. NPF-76 and Facility Operating License No. NPF-80, issued to STP Nuclear Operating Company (the licensee), for operation of the South Texas Project (STP), Units 1 and 2, located in Matagorda County, Texas.

Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt STP Nuclear Operating Company from the requirements of Title 10 of the Code of Federal Regulations (10 CFR) § 50.60, which requires all power reactors to meet the fracture toughness and material surveillance program requirements for the reactor coolant pressure boundary set forth in appendices G and H to 10 CFR part 50.

The proposed exemption would allow STP Nuclear Operating Company to apply American Society of Mechanical Engineers (ASME) Code Case N-514 for determining STP's cold overpressurization mitigation system (COMS) pressure setpoint.

The proposed action is in accordance with the licensee's application for exemption dated March 18, 1999.

The Need for the Proposed Action

The proposed exemption is needed to support an amendment to the STP Technical Specifications which will revise the heatup, cooldown, and COMS curves. The use of ASME Code Case N-514 would allow an increased operating band for system makeup and pressure control.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that the proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not involve any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the Commission concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the "Final Environmental Statement Related to the Operation of South Texas Project, Units 1 and 2," dated August 1996, in NUREG-1171.

Agencies and Persons Consulted

In accordance with its stated policy, on March 30, 1999, the staff consulted with Texas State Official, Mr. Arthur C. Tate of the Texas Department of Health regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated March 18, 1999, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Wharton County Junior College, J.M. Hodges Learning Center, 911 Boling Highway, Wharton, Texas.

Dated at Rockville, Md., this 27th day of April 1999.

For the Nuclear Regulatory Commission.

Robert A. Gramm,

Chief, Section 1, Project Directorate IV & Decommissioning, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-11025 Filed 4-30-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Subcommittee Meeting on Thermal-Hydraulic Phenomena; Notice of Meeting

The ACRS Subcommittee on Thermal-Hydraulic Phenomena will hold a meeting on May 26, 1999, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

Portions of this meeting may be closed to public attendance to discuss Electric Power Research Institute (EPRI) proprietary information pursuant to 5 U.S.C. 552b(c)(4).

The agenda for the subject meeting shall be as follows:

Wednesday, May 26, 1999—8:30 a.m. until the Conclusion of Business

The Subcommittee will review the: (1) Proposed resolution of Generic Safety Issue 23, "Reactor Coolant Pump Seal Failures," (2) proposed rule to revise Appendix K to 10 CFR 50.46 to allow

small, cost beneficial power uprates, and (3) status of the EPRI RETRAN-3D transient thermal-hydraulic code review and proposed ACRS Structured Discussion on development of code review guidelines. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman. Written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, and other interested persons regarding this review. Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the scheduling of sessions which are open to the public, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACRS staff engineer, Mr. Paul A. Boehnert (telephone 301/415-8065) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: April 27, 1999.

Richard P. Savio,

Associate Director for Technical Support, ACRS/ACNW.

[FR Doc. 99-11019 Filed 4-30-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Subcommittee Meeting on Severe Accident Management; Meeting

The ACRS Subcommittee on Severe Accident Management will hold a meeting on May 27, 1999, Room T-2B3, 11545 Rockville Pike, Rockville, Maryland.

The agenda for the subject meeting shall be as follows:

Thursday, May 27, 1999—8:30 a.m. until the conclusion of business.

The Subcommittee will review the application of the Southern California Edison Company for an exemption to the hydrogen control requirements for the San Onofre Nuclear Generating Station. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman. Written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the Southern California Edison Company, the NRC staff, and other interested persons regarding this review. Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the scheduling of sessions which are open to the public, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting the cognizant ACRS staff engineer, Mr. Paul A. Boehnert (telephone 301/415-8065) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are