

30, 1998. Applicants selected will not have to recompetite for funding for PY 1998 (July 1, 1998 to June 30, 1999) if the grant recipient has met all applicable regulatory requirements, has performed satisfactorily under the terms of its existing grant for PY 1997, submits an acceptable training plan or PY 1998, and funds are available.

(b) In the event that no grant applications will be received for a specific State or sub-State area or those received are deemed to be unacceptable, or where a grant agreement is not successfully negotiated, DOL may give the Governor first right to submit an acceptable application pursuant to the precondition for Grant Application and Responsibility Review tests at 20 CFR 633.201 and 633.204, respectively. Should the Governor not accept the offer within 15 days after being notified, the Department may then: (1) designate another organization or organizations, (2) reopen the area for competitive bidding, or (3) use the allocated funds for national account activities.

(c) An applicant whose grant application is not selected by DOL to receive JTPA section 402 funds will be notified in writing.

(d) Any applicant whose grant application is denied in whole or part by DOL will be advised of its appeal rights.

Signed at Washington, DC, this 4th day of February, 1997.

James DeLuca,

Grant Officer, Division of Acquisition and Assistance.

[FR Doc. 97-3347 Filed 2-10-97; 8:45 am]

BILLING CODE 4510-30-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-327 AND 50-328]

Sequoyah Nuclear Plant, Units 1 and 2; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License No. DPR-77 and DPR-79 issued to the Tennessee Valley Authority (the licensee) for operation of the Sequoyah Nuclear Plant, Units 1 and 2, located in Soddy Daisy, Tennessee.

The proposed amendments would permanently incorporate requirements associated with steam generator tube inspections and repair in the Sequoyah

Nuclear Plant, Units 1 and 2 Technical Specifications (TS). The new requirements establish alternate steam generator tube plugging criteria (APC) at the tube support plate intersections. These revised criteria, based on NRC Generic Letter 95-05, were incorporated into the TS by previous amendments to the operating licenses but only for Operating Cycle 8. The proposed amendments would remove the reference to Cycle 8, thereby making the requirements applicable to all future operating cycles.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

TVA has evaluated the proposed technical specification (TS) change and has determined that it does not represent a significant hazards consideration based on criteria established in 10 CFR 50.92(c). Operation of Sequoyah Nuclear Plant (SQN) in accordance with the proposed amendment will not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed TS change revises the SQN steam generator (S/G) Specification 3/4.4.5 to remove footnotes that limit the application of the alternate plugging criteria (APC) to Cycle 8 operation only. In addition, SQN TS 3.4.6.2, "Operational Leakage," contains a similar footnote that limits application of S/G APC to Cycle 8 operation only. The removal of these footnotes allows TVA to apply APC to SQN S/Gs beyond Cycle 8 operation. TVA's proposed change is based on resolution of the industry issues concerning [eddy current test] probe wear and probe variability. APC was applied to the SQN S/Gs during the Cycle 7 refueling outages for Units 1 and 2.

The proposed changes provide TS requirements that are consistent with the guidance of NRC GL [Generic Letter] 95-05. This change does not involve a physical modification to the plant or affect any setpoints. Accordingly, the proposed changes

do not involve an increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any previously analyzed.

The proposed changes provide TS requirements for SQN S/Gs that are consistent with the guidance provided in GL 95-05. No new event initiator has been created, nor has any hardware been changed. This change does not involve a physical change to SQN S/Gs or any other system. Therefore, the proposed change will not create the possibility of a new or different kind of accident from any previously analyzed.

3. Involve a significant reduction in a margin of safety.

TVA's proposed change allows application of APC for SQN S/Gs to extend beyond Cycle 8 of operation. This change continues to provide requirements that maintain structural integrity of SQN S/G tubes during normal operating, transient, and postulated accident conditions. This change does not involve a setpoint change or physical modification to the plant. Accordingly, the margin of safety has not been reduced.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the Federal Register a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules Review and Directives Branch, Division of Freedom of Information and Publications

Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By March 13, 1997, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Chattanooga-Hamilton County Library, 1001 Broad Street, Chattanooga, Tennessee 37402. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible

effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the

Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1 (800) 248-5100 (in Missouri 1 (800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Frederick J. Hebdon: Petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to General Counsel, Tennessee Valley Authority, ET 11H 400 West Summit Hill Drive, Knoxville, Tennessee 37902, attorney for the licensee.

Untimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1) (i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated October 18, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Chattanooga-Hamilton County Library, 1001 Broad Street, Chattanooga, Tennessee 37402.

Dated at Rockville, Maryland, this 5th day of February 1997.

Ronald W. Hernan,

Senior Project Manager, Project Directorate II-3, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 97-3321 Filed 2-10-97; 8:45 am]

BILLING CODE 7590-01-P

[Docket 70-7002]

Notice of Amendment to Certificate of Compliance GDP-2 for the U.S. Enrichment Corporation, Portsmouth Gaseous Diffusion Plant, Portsmouth, OH

The Director, Office of Nuclear Material Safety and Safeguards, has made a determination that the following amendment request is not significant in accordance with 10 CFR 76.45. In making that determination, the staff concluded that: (1) There is no change in the types or significant increase in the amounts of any effluents that may be released offsite; (2) there is no significant increase in individual or cumulative occupational radiation exposure; (3) there is no significant construction impact; (4) there is no significant increase in the potential for, or radiological or chemical consequences from, previously analyzed accidents; (5) the proposed changes do not result in the possibility of a new or different kind of accident; (6) there is no significant reduction in any margin of safety; and (7) the proposed changes will not result in an overall decrease in the effectiveness of the plant's safety, safeguards, or security programs. The basis for this determination for the amendment request is described below.

The NRC staff has reviewed the certificate amendment application and concluded that it provides reasonable assurance of adequate safety, safeguards, and security and compliance with NRC requirements. Therefore, the Director, Office of Nuclear Material

Safety and Safeguards, is prepared to issue an amendment to the Certificate of Compliance for the Portsmouth Gaseous Diffusion Plant (PORTS). The staff has prepared a Compliance Evaluation Report which provides details of the staff's evaluation.

The NRC staff has determined that this amendment satisfies the criteria for a categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for this amendment.

USEC or any person whose interest may be affected may file a petition, not exceeding 30 pages, requesting review

of the Director's Decision. The petition must be filed with the Commission not later than 15 days after publication of this Federal Register Notice. A petition for review of the Director's Decision shall set forth with particularity the interest of the petitioner and how that interest may be affected by the results of the decision. The petition should specifically explain the reasons why review of the Decision should be permitted with particular reference to the following factors: (1) The interest of the petitioner; (2) how that interest may be affected by the Decision, including the reasons why the petitioner should be permitted a review of the Decision; and (3) the petitioner's areas of concern about the activity that is the subject matter of the Decision. Any person described in this paragraph (USEC or any person who filed a petition) may file a response to any petition for review, not to exceed 30 pages, within 10 days after filing of the petition. If no petition is received within the designated 15-day period, the Director will issue the final amendment to the Certificate of Compliance without further delay. If a petition for review is received, the decision on the amendment application will become final in 60 days, unless the Commission grants the petition for review or otherwise acts within 60 days after publication of this Federal Register Notice.

A petition for review must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, by the above date.

For further details with respect to the action see: (1) The application for amendment and (2) the Commission's Compliance Evaluation Report. These items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, and at the Local Public Document Room.

Date of amendment request: November 8, 1996, as modified by USEC responses dated December 13, 1996, and January 16, 1997, to NRC requests for additional information dated November 29, 1996, and December 31, 1996, respectively.

Brief description of amendment: The amendment changes the Technical Safety Requirement (TSR) Standby Operational Mode definition for the UF6 Withdrawal Stations by allowing the compression loop vent path to the cascade to be open. It should be noted

that venting of the Withdrawal Station compression loop to the cascade is routinely done at PORTS. However, accounting for this procedure was inadvertently left out of the Standby Operational Mode definition by USEC from its proposed TSRs which have been approved by the NRC.

Basis for finding of no significance:

1. The proposed amendment will not result in a change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed change to TSR 2.5.1 permits evacuating UF6 from the compression loop in the UF6 withdrawal station to the cascade, which acts as a low pressure sink, in the Standby Operational Mode. This change will not result in significantly increasing the potential for unconfinement of UF6 which could lead to an increase in effluents that may be released offsite since it only involves venting of UF6 from one portion of process piping, which confines UF6 in the Withdrawal Station, to another portion of process piping which confines UF6 in the enrichment cascade. Confinement of UF6 within the cascade is primarily provided by maintaining the cell high-side (compressor discharge) gas pressure below 25 psia (TSR 2.2.3.13) and by applying appropriate quality assurance requirements to process gas piping and equipment (Safety Analysis Report Section 3.8.2.2). Therefore, this TSR amendment will not result in significant amounts of effluents that may be released offsite.

2. The proposed amendment will not result in a significant increase in individual or cumulative occupational radiation exposure.

Evacuating UF6 from the compression loop to the cascade in the Standby Operational Mode will not significantly impart additional occupational radiation exposure. The cascade or the withdrawal loops do not result in significant occupational radiation exposures. Some of the reasons being that: (1) The occupancy factor is low, (2) distance from the source is generally high, (3) significant shielding is provided by piping and equipment, (4) depleted and low enriched uranium has low specific activities and are also comparatively low gamma radiation emitters, (5) most of the uranium is in gaseous form (low density), and (6) UF6 is confined within quality controlled equipment and piping. Therefore, any transfer of confined UF6 from the withdrawal station to the cascade would not measurably modify individual or