required to prepare or cause to be prepared environmental documents relating to actions by the agency that have significant impacts on the environment. The Commission believes that the NEPA process will be triggered when a tribe and management contractor seek approval of a management contract under 25 CFR 533.

Respondents: Applicants seeking approval of a management contract and/or third party contractor.

Estimated Number of Respondents: 11.

Estimated Annual Responses: 11.

Estimated Annual Burden Hours: 5000.

Estimated Burden Hours Per Response: 455.

Title: Annual Fees Payable by Class II Gaming Operations.

OMB Number: 3141–0007.

Abstract: The IGRA authorizes the NIGC to establish a schedule of fees to be paid to the Commission by each class II gaming operation regulated by the IGRA. Fees are computed using rates set by the NIGC and the assessable gross revenues of each gaming operation. The total of all fees assessed annually cannot exceed $1,500,000. The required information is needed for the NIGC to both set and adjust rates and to support the computation of fees paid by each gaming operation.

Respondents: Class II gaming operations.

Estimated Number of Respondents: 201.

Estimated Annual Responses: 404.

Estimated Burden Hours Per Response: 5.

FOR COPIES AND FURTHER INFORMATION CONTACT: Copies of documents submitted to OMB may be obtained from the National Indian Gaming Commission, 1441 L Street NW, Suite 9100, Washington, DC 20005.

Tadd M. Johnson,

Chairman, National Indian Gaming Commission.

[FR Doc. 97–28876 Filed 10–30–97; 8:45 am]

BILLING CODE 7567–01–M

NUCLEAR REGULATORY COMMISSION

Application for a License To Import Nuclear Waste

Pursuant to 10 CFR 110.70(b) ''Public notice of receipt of an application'', please take notice that the Nuclear Regulatory Commission has received the folowing application for an import license. Copies of the application are on file in the Nuclear Regulatory Commission's Public Document Room located at 2120 L Street, N.W., Washington, D.C..

A request for a hearing or petition for leave to intervene may be filed within 30 days after publication of this notice in the **Federal Register**. Any request for hearing or petition for leave to intervene shall be served by the requestor or petitioner upon the applicant, the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington D.C. 20555; the Secretary, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; and the Executive Secretary, U.S. Department of State, Washington, D.C. 20520.

The information concerning the application follows.

NRC IMPORT LICENSE APPLICATION

Name of applicant, date of application, date received, application no.	Description of material			Country of origin
	Material type	Total qty	End use	
Chem-Nuclear Systems, October 14, 1997, October 20, 1997, IW005.	Contaminated Condenser tubes and tube plates.	1.4 million	Decontamination and recycling	Taiwan.

For the Nuclear Regulatory Commission.

Dated this 24th day of October 1997 at Rockville, Maryland.

Ronald D. Hauber,

Director, Division of Nonproliferation, Exports and Multilateral Relations, Office of International Programs.

[FR Doc. 97–28896 Filed 10–30–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. STN 50–457]

Commonwealth Edison Company; Braidwood Station, Unit 2 Environmental Assessment and Finding Of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of its regulations for Facility Operating License No. NPF–77, issued to Commonwealth Edison Company, (ComEd, the licensee), for operation of the Braidwood Station, Unit 2, located in Will County, Illinois.

Environmental Assessment

Identification of the Proposed Action

The proposed action would permit the licensee to use the alternate methodology in American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Case N–514, ''Low Temperature Overpressure Protection,'' to determine the low temperature overpressure protection (LTOP) system setpoints. By application dated November 30, 1994, as supplemented by letter dated May 11, 1995, the licensee requested an exemption from certain requirements of 10 CFR 50.60, ''Acceptance Criteria for Fracture Prevention Measures for Lightwater Nuclear Power Reactors for Normal Operation.'' The exemption would allow application of an alternate methodology to determine the LTOP system setpoints for Braidwood, Unit 2. The proposed alternate methodology is consistent with guidelines developed by the ASME Working Group on Operating Plant Criteria to define pressure limits during LTOP events that avoid certain unnecessary operational restrictions, provide adequate margins against failure of the reactor pressure vessel, and reduce the potential for unnecessary activation of pressure relieving devices used for LTOP. These guidelines have been incorporated into the 1993 Addenda to the ASME Code, Section XI, Appendix G. However, 10 CFR 50.55a, ''Codes and Standards,'' has not been updated to reflect the acceptability of the 1993 Addenda to the ASME Code.

The Need for the Proposed Action

Pursuant to 10 CFR 50.60, all lightwater nuclear power reactors must meet the fracture toughness requirements for the reactor coolant pressure boundary as set forth in 10 CFR

Part 50, Appendix G. Appendix G of 10 CFR Part 50 defines pressure-temperature (P–T) limits during any condition of normal operation, including anticipated operational occurrences and system hydrostatic tests to which the pressure boundary may be subjected over its service lifetime, and specifies that these P–T limits must be at least as conservative as the limits obtained by following the methods of analysis and the margins of safety of the ASME Code, Section XI, Appendix G. It is required in 10 CFR 50.55a that any reference to the ASME Code, Section XI, in 10 CFR Part 50 refers to addenda through the 1988 Addenda and editions through the 1989 Edition of the Code unless otherwise noted. It is specified in 10 CFR 50.60(b) that alternatives to the described requirements in 10 CFR Part 50, Appendix G, may be used when an exemption is granted by the Commission under 10 CFR 50.12.

To prevent transients that would produce excursions exceeding the P–T limits while the reactor is operating at low temperatures, the licensee installed the LTOP system, which includes pressure relieving devices called power-operated relief valves (PORVs). The PORVs prevent the pressure in the reactor vessel from exceeding the P–T limits. However, to prevent the PORV from lifting as a result of normal operating pressure surges, some margin is needed between the normal operating pressure and the PORV setpoint. In addition, normal operating pressure must be high enough to prevent damage to reactor coolant pumps that may result from cavitation or inadequate differential pressure across the pump seals. Hence, the licensee must operate the plant in a pressure window that is defined as the difference between the minimum pressure required for reactor coolant pumps and the operating margin to prevent lifting of the PORVs. When instrument uncertainty is considered, the operating window is small and presents difficulties for plant operation.

To meet the 10 CFR Part 50, Appendix G, P–T limits, the PORVs would be set to open at a pressure very close to the normal pressure inside the reactor. With the PORV setpoint close to the normal operating pressure, minor pressure perturbations that typically occur in the reactor could cause the PORVs to open. This is undesirable from the safety perspective because after every PORV opening there is some concern that the PORV may not reclose. A stuck open PORV would continue to discharge primary coolant and reduce reactor pressure until the discharge pathway was closed by operator action.

The licensee requested use of the ASME Code Case N–514, ''Low Temperature Overpressure Protection,'' for the determination of the PORV setpoints. This code case would permit a slightly higher PORV setpoint during low-temperature shutdown conditions. This would reduce the likelihood for inadvertent opening of the PORVs.

Appendix G of the ASME Code requires that the P–T limits be calculated: (a) using a safety factor of two on the principal membrane (pressure) stresses, (b) assuming a flaw at the surface with a depth of one quarter (¼) of the vessel wall thickness and a length of six (6) times its depth, and (c) using a conservative fracture toughness curve that is based on the lower bound of static, dynamic, and crack arrest fracture toughness tests on material similar to the Braidwood reactor vessel material.

ASME Code Case N–514 requires that the system pressure is maintained below the P–T limits during normal operation, but allows the pressure that may occur with the activation of pressure relieving devices (PORVs) to exceed the P–T limits, provided acceptable margins are maintained during these events. This approach protects the pressure vessel from LTOP events, and maintains the Technical Specification P-T limits applicable for normal heatup and cooldown in accordance with 10 CFR Part 50, Appendix G, and Sections III and XI of the ASME Code.

In determining the PORV setpoint for LTOP events, the licensee proposed to use the safety margins of ASME Code Case N–514. This alternate methodology allows determination of the setpoint for LTOP events such that the maximum pressure in the vessel will not exceed 110 percent of the P–T limits. This results in a safety factor of 1.8 on the principal membrane stresses. All other factors, including the assumed flaw size and fracture toughness, remain the same. Although this methodology would reduce the safety factor on the principal membrane stresses, use of the proposed criteria will provide adequate margins of safety for the reactor vessel during LTOP events.

Use of the Code Case N–514 safety margins will reduce operational challenges during low temperature, low pressure operations. In terms of overall safety, the safety benefits derived from simplified operations and the reduced potential for undesirable opening of the PORVs will more than offset the reduction of the principal membrane safety factor. Reduced operational challenges will reduce the potential for undesirable impacts to the environment.

Environmental Impacts of the Proposed Action

The proposed action involves features located entirely within the protected area as defined in 10 CFR Part 20.

The proposed action will not result in an increase in the probability or consequences of accidents or result in a change in occupational or offsite dose. Therefore, there are no radiological impacts associated with the proposed action.

The proposed action will not result in a change in nonradiological plant effluent and will have no other nonradiological environmental impact.

Accordingly, the Commission concludes that there are no environmental impacts associated with this action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Braidwood Station.

Agencies and Persons Consulted

In accordance with its stated policy, on October 22, 1997, the staff consulted with the Illinois State official, Frank Niziolek of the Illinois Department of Nuclear Safety, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated November 30, 1994, as supplemented by letter dated May 11, 1995, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC,

and at the local public document room located at the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481.

Dated at Rockville, Maryland, this 23rd day of October 1997.

For the Nuclear Regulatory Commission.

George F. Dick, Jr.,

Senior Project Manager, Project Directorate III–2, Division of Reactor Projects III/IV, Office of Nuclear Reactor Regulation.

[FR Doc. 97–28881 Filed 10–30–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Meeting of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment; Revised

A meeting of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment scheduled to be held on November 13-14, 1997 has been rescheduled for *Wednesday, November 12, 1997 and Thursday, November 13, 1997*, Room T–2B3, 11545 Rockville Pike, Rockville, Maryland. Notice of this meeting was published in the **Federal Register** on Friday, October 24, 1997 (62 FR 55435).

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Wednesday, November 12, 1997—8:30 a.m. until the conclusion of business

Thursday, November 13, 1997—8:30 a.m. until the conclusion of business

The Subcommittee will review the proposed final Standard Review Plan (SRP) Chapter 19 and associated Regulatory Guide DG-1061(General Guidance) for risk-informed, performance-based regulation. The Subcommittee will continue its review of the matter included in the Staff Requirements Memorandum dated May 27, 1997, regarding the use of uncertainty versus point values in the PRA-related regulatory decisionmaking process. The Subcommittee will discuss policy issues related to performance-based regulation, including industry initiatives in this area. All other items regarding this meeting remain the same as announced in the **Federal Register** published on Friday, October 24, 1997 (62 FR 55435).

Further information regarding this meeting can be obtained by contacting the cognizant ACRS staff engineer, Mr. Michael T. Markley (telephone 301/415–6885) between 7:30 a.m. and 4:15 p.m. (EST).

Dated: October 27, 1997.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 97–28895 Filed 10–30–97; 8:45 am]

BILLING CODE 7590–01–P

SMALL BUSINESS ADMINISTRATION

[License No. 03/73–0212]

CEO Venture Fund III, L.P.; Notice of Issuance of a Small Business Investment Company License

On December 26, 1996, an application was filed by CEO Venture Fund III, L.P., at 2000 Technology Drive, Suite 160, Pittsburgh, Pennsylvania 15219–3109, with the Small Business Administration (SBA) pursuant to Section 107.300 of the Regulations governing small business investment companies (13 C.F.R. 107.300 (1996)) for a license to operate as a small business investment company.

Notice is hereby given that, pursuant to Section 301(c) of the Small Business Investment Act of 1958, as amended, after having considered the application and all other pertinent information, SBA issued License No. 03/73–0212 on August 1, 1997, to CEO Venture Fund III, L.P. to operate as a small business investment company.

(Catalog of Federal Domestic Assistance Program No. 59.011, Small Business Investment Companies)

Dated: October 22, 1997.

Don A. Christensen,

Associate Administrator for Investment.

[FR Doc. 97–28907 Filed 10–30–97; 8:45 am]

BILLING CODE 8025–01–P

SMALL BUSINESS ADMINISTRATION

Revocation of License of Small Business Investment Company

Pursuant to the authority granted to the United States Small Business Administration by the Final Order of the United States District Court for the Eastern District of New York, dated September 23, 1997, the United States Small Business Administration hereby revokes the license of S & S Venture Associates, Ltd., a New York corporation, to function as a small business investment company under the Small Business Investment Company License No. 02/02–0383 issued to S & S Venture Associates, Ltd. on April 25, 1980 and said license is hereby declared null and void as of October 21, 1997.

United States Small Business Administration.

Dated: October 21, 1997.

Don A. Christensen,

Associate Administrator for Investment.

[FR Doc. 97–28905 Filed 10–30–97; 8:45 am]

BILLING CODE 8025–01–P

SMALL BUSINESS ADMINISTRATION

Revocation of License of Small Business Investment Company

Pursuant to the authority granted to the United States Small Business Administration by the Final Order of the United States District Court for the Eastern District of Pennsylvania, entered August 15, 1997, the United States Small Business Administration hereby revokes the license of Salween Financial Services, Inc., a Pennsylvania corporation, to function as a small business investment company under the Small Business Investment Company License No. 03/03–5157 issued to Salween Financial Services, Inc. on July 1, 1983 and said license is hereby declared null and void as of October 15, 1997.

United States Small Business Administration.

Dated: October 15, 1997.

Don A. Christensen,

Associate Administrator for Investment.

[FR Doc. 97–28906 Filed 10–30–97; 8:45 am]

BILLING CODE 8025–01–P

SOCIAL SECURITY ADMINISTRATION

Information Collection Activities: Proposed Collection Requests and Comment Requests

This notice lists information collection packages that will require submission to the Office of Management and Budget (OMB), as well as information collection packages submitted to OMB for clearance, in compliance with PL. 104–13 effective October 1, 1995, The Paperwork Reduction Act of 1995.

I. The information collection(s) listed below require(s) extension(s) of the current OMB approval(s) or are proposed new collection(s):

1. Request to be Selected as Payee—0960–0014. The information collected on Form SSA–11–BK is used to determine the proper payee for a Social Security beneficiary, and it is designed to aid in the investigation of a payee applicant. The form will establish the applicant's relationship to the beneficiary, the justification, the concern for the beneficiary and the manner in which the benefits will be used. The respondents are applicants for