criticality accident monitoring system in each area where such material is handled, used, or stored. Subsection a(2) of 10 CFR 70.24 specifies detection and sensitivity requirements that these monitors must meet. Subsection a(2) also specifies that all areas subject to criticality accident monitoring must be covered by monitoring devices.

Subsection (a)(3) of 10 CFR 70.24 requires licensees to maintain emergency procedures for each area in which this licensed special nuclear material is handled, used, or stored and provides (1) that the procedures ensure that all personnel withdraw to an area of safety upon the sounding of a criticality accident monitor alarm, (2) that the procedures must include drills to familiarize personnel with the evacuation plan, and (3) that the procedures designate responsible individuals for determining the cause of the alarm and placement of radiation survey instruments in accessible locations for use in such an emergency. Subsection (b)(1) of 10 CFR 70.24 requires licensees to have a means to identify quickly personnel who have received a dose of 10 rads or more. Subsection (b)(2) of 10 CFR 70.24 requires licensees to maintain personnel decontamination facilities, to maintain arrangements for a physician and other medical personnel qualified to handle radiation emergencies, and to maintain arrangements for the transportation of contaminated individuals to treatment facilities outside the site boundary. Subsection (c) of 10 CFR 70.24 exempts Part 50 licensees from the requirements of subsection (b) of 10 CFR 70.24 for special nuclear material used or to be used in the reactor. Subsection (d) of 10 CFR 70.24 states that any licensee who believes that there is good cause why he should be granted an exemption from all or part of 10 CFR 70.24 may apply to the Commission for such an exemption and shall specify the reasons for the relief requested.

III

The special nuclear material that could be assembled into a critical mass at Point Beach Nuclear Plant is in the form of nuclear fuel; the quantity of special nuclear material other than fuel that is stored on site is small enough to preclude achieving a critical mass. The Commission technical staff has evaluated the possibility of an inadvertent criticality of the nuclear fuel at Point Beach Nuclear Plant and has determined that such an accident cannot occur if the licensee meets the following seven criteria:

1. Only one new fuel assembly is allowed out of a shipping cask or storage rack at one time.

2. The k-effective does not exceed 0.95, at a 95% probability, 95% confidence level in the event that the fresh fuel storage racks are filled with fuel of the maximum permissible U-235 enrichment and flooded with pure water.

3. If optimum moderation occurs at low moderator density, then the k-effective does not exceed 0.98, at a 95% probability, 95% confidence level in the event that the fresh fuel storage racks are filled with fuel of the maximum permissible U-235 enrichment and flooded with moderator at the density corresponding to optimum moderation.

4. The k-effective does not exceed 0.95, at a 95% probability, 95% confidence level in the event that the spent fuel storage racks are filled with fuel of the maximum permissible U-235 enrichment and flooded with pure water.

5. The quantity of forms of special nuclear material, other than nuclear fuel, that are stored on site in any given area is less than the quantity necessary for a critical mass.

6. Radiation monitors, as required by Point Beach Nuclear Plant General Design Criterion 18 which is analogous to 10 CFR part 50, Appendix A, General Design Criterion 63, are provided in fuel storage and handling areas to detect excessive radiation levels and to initiate appropriate safety actions.

7. The maximum nominal U-235 enrichment is limited to 5 weight percent.

By letter dated June 7, 1997, Wisconsin Electric Power Company requested an exemption from 10 CFR 70.24. In this exemption request the licensee addressed the seven criteria given above. The Commission technical staff has reviewed the licensee's submittal and has determined that Point Beach Nuclear Plant, Units 1 and 2, meets the criteria for prevention of inadvertent criticality; therefore, the staff has determined that it is extremely unlikely for an inadvertent criticality to occur in special nuclear materials handling or storage areas at Point Beach Nuclear Plant.

The purpose of the criticality monitors required by 10 CFR 70.24 is to ensure that, if a criticality were to occur during the handling of special nuclear material, personnel would be alerted to that fact and would take appropriate action. The staff has determined that it is extremely unlikely that such an accident could occur; furthermore, the licensee has radiation monitors, as required by Point Beach Nuclear Plant's General Design Criterion 18, in fuel storage and handling areas. Specifically, Point Beach Nuclear Plant's General Design Criterion 18 requires ''Monitoring and alarm instrumentation shall be provided for fuel and waste storage and associated handling areas for conditions that might result in loss of capability to remove decay heat and to detect excessive radiation levels.'' These monitors will alert personnel to excessive radiation levels and allow them to initiate appropriate safety actions. The low probability of an inadvertent criticality together with the licensee's adherence to Point Beach Nuclear Plant's General Design Criterion 18 constitute good cause for granting an exemption to the requirements of 10 CFR 70.24.

IV

The Commission has determined that, pursuant to 10 CFR 70.14, this exemption is authorized by law, will not endanger life or property or the common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants the Wisconsin Electric Power Company an exemption from the requirements of 10 CFR 70.24 for the Point Beach Nuclear Plant.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the quality of the human environment (62 FR 45883).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 6th day of October 1997.

For the Nuclear Regulatory Commission.

Samuel J. Collins,

Director, Office of Nuclear Reactor Regulation.

[FR Doc. 97–27085 Filed 10–10–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps; Issue

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of issuance.

SUMMARY: The Nuclear Regulatory Commission (NRC) has issued Generic Letter 97–04 to all holders of operating licenses for nuclear power plants, except those who have permanently ceased operations and have certified that fuel has been permanently removed

from the reactor vessel, to request that addressees submit information necessary to confirm the adequacy of the net positive suction head available for emergency core cooling (including core spray and decay heat removal) and containment heat removal pumps.

The generic letter is available in the NRC Public Document Room under accession number 9710060324.

DATES: The generic letter was issued on October 7, 1997.

ADDRESSES: Not applicable.

FOR FURTHER INFORMATION CONTACT: William O. Long at (301) 415–3026.

SUPPLEMENTARY INFORMATION: This generic letter only requests information from addressees under the provisions of Section 182a of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.54(f). The requested information will enable the staff to determine whether the NPSH analyses of addressees conform with the current licensing basis for their respective facilities, including the licensing safety analyses and the principal design criteria which require and/or commit that safety-related components and systems be provided to mitigate the consequences of design-basis accidents.

New NPSH analyses are neither requested nor required to be performed to respond to this information request. However, new NPSH analyses may be warranted if an addressee determines that changes in plant design or procedures have occurred which may have reduced the available NPSH. In such cases, each affected addressee must take appropriate corrective action to restore its facility to compliance, in accordance with the requirements stated in Appendix B to 10 CFR Part 50.

Dated at Rockville, Maryland, this 7th day of October 1997.

For the Nuclear Regulatory Commission.

Jack W. Roe,

Acting Director, Division of Reactor Program Management, Office of Nuclear Reactor Regulation.

[FR Doc. 97–27083 Filed 10–10–97; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards Meeting of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment; Notice of Meeting

The ACRS Subcommittee on Reliability and Probabilistic Risk Assessment will hold a meeting on October 21 (Room T–2B3) and 22 (Room T–2B1), 1997, 11545 Rockville Pike, Rockville, Maryland.

The entire meeting will be open to public attendance.

The agenda for the subject meeting shall be as follows:

Tuesday, October 21, 1997—8:30 a.m. until the conclusion of business
Wednesday, October 22, 1997—8:30 a.m. until 12:00 Noon

The Subcommittee will continue its review of matters related to the Staff Requirements Memorandum dated May 27, 1997, regarding the use of uncertainty versus point values in the PRA-related regulatory decisionmaking process. The Subcommittee will also review the proposed final Standard Review Plan (SRP) sections and Regulatory Guides for risk-informed, performance-based regulation. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, its consultants, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting the cognizant ACRS staff engineer, Mr. Michael T. Markley (telephone 301/415–6885) between 7:30 a.m. and 4:15 p.m. (EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes to the agenda, etc., that may have occurred.

Dated: October 7, 1997.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 97–27081 Filed 10–10–97; 8:45 am]

BILLING CODE 7590–01–P

OFFICE OF PERSONNEL MANAGEMENT

Proposed Collection; Comment Request for Reclearance of an Information Collection: Standard Forms 2803 and 3108

AGENCY: Office of Personnel Management.

ACTION: Notice.

SUMMARY: In accordance with the Paperwork Reduction Act of 1995 (Pub. L. 104–13, May 22, 1995), this notice announces that the Office of Personnel Management intends to submit to the Office of Management and Budget a request for reclearance of an information collection. Form 2803, Application to Make Deposit or Redeposit (CSRS), and SF 3108, Application to Make Service Credit Payment for Civilian Service (FERS), are applications to make payment used by persons who are eligible to pay for Federal service which was not subject to retirement deductions and/or for Federal service which was subject to retirement deductions which were subsequently refunded to the applicant.

There are approximately 520 respondents for SF 2803 and 260 respondents for SF 3108. It takes approximately 30 minutes to complete SF 2803 and 30 minutes to complete SF 3108. The hourly annual burden for SF 2803 is 260 and for SF 3108 is 130 for a total annual burden of 390 hours.

Comments are particularly invited on:

—Whether this collection of information is necessary for the proper performance of functions of the Office of Personnel Management, and whether it will have practical utility;

—Whether our estimate of the public burden of this collection is accurate, and based on valid assumptions and methodology; and

—Ways in which we can minimize the burden of the collection of information on those who are to respond, through use of the appropriate technological collection techniques or other forms of information technology.

For copies of this proposal, contact Jim Farron on (202) 418–3208, or E-mail to jmfarron@opm.gov