

FOR FURTHER INFORMATION CONTACT:
Becky Baker, Secretary of the Board,
Telephone 703-518-6304.

Becky Baker,

Secretary of the Board.

[FR Doc. 97-19042 Filed 7-15-97; 4:31 pm]

BILLING CODE 7535-01-M

NATIONAL CREDIT UNION ADMINISTRATION

Sunshine Act Meeting

TIME AND DATE: 10:30 a.m., Tuesday, July 22, 1997.

PLACE: Board Room, 7th Floor, Room 7047, 1775 Duke Street, Alexandria, VA 22314-3428.

STATUS: Closed.

MATTERS TO BE CONSIDERED:

1. Administrative Actions under Section 205 of the Federal Credit Union Act and Section 708b of NCUA's Rules and Regulations. Closed pursuant to exemption (8).

2. Administrative Action under Sections 125 and 206 of the Federal Credit Union Act and Section 708b of NCUA's Rules and Regulations. Closed pursuant to exemption (8).

3. Proposed National Small Credit Union Development Program (NSCUDP). Closed pursuant to exemptions (2), (6) and (8).

4. Personal Actions. Closed pursuant to exemptions (2) and (6).

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[FR Doc. 97-19043 Filed 7-15-97; 4:31 pm]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 30-30691-CivP; ASLBP No. 97-730-02-CivP]

Barnett Industrial X-Ray, Inc.; Establishment of Atomic Safety and Licensing Board

Pursuant to delegation by the Commission dated December 29, 1972, published in the **Federal Register**, 37 FR 28710 (1972), and Sections 2.105, 2.700, 2.702, 2.714, 2.714a, 2.717, 2.721, and 2.772(j) of the Commission's Regulations, all as amended, an Atomic Safety and Licensing Board is being established to preside over the following proceeding.

Barnett Industrial X-Ray, Inc.; Order Imposing Civil Monetary Penalty

This Board is being established pursuant to the request of Barnett Industrial X-Ray, Inc. for an enforcement hearing. The hearing request was made in response to an Order issued by the Director, Office of Enforcement, dated May 23, 1997, entitled "Order Imposing Civil Monetary Penalty" (62 FR 30346, June 3, 1997).

The Board is comprised of the following administrative judges:

B. Paul Cotter, Jr., Chairman, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555

Dr. Richard F. Cole, Atomic Safety and Licensing Board Panel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555

Dr. Thomas S. Elleman, 704 Davidson Street, Raleigh, NC 27609

All correspondence, documents and other materials shall be filed with the Judges in accordance with 10 CFR § 2.701. /s/ B. Paul Cotter, Jr.

Issued at Rockville, Maryland, this 14th day of July 1997.

B. Paul Cotter, Jr.,

Chief Administrative Judge, Atomic Safety and Licensing Board Panel.

[FR Doc. 97-18968 Filed 7-17-97; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Nuclear Byproduct Material Risk Review Group

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Establishment of working group and notice of meeting.

SUMMARY: The Nuclear Byproduct Material Risk Review Group, consisting of representatives from the U.S. Nuclear Regulatory Commission and an Agreement State, has been formed in response to Commission direction in a Staff Requirements Memorandum (SRM) dated April 15, 1997. The SRM instructed the staff to reexamine the applicability of risk-informed, performance-based or risk-informed, less prescriptive approaches to regulation of nuclear material. The working group will meet at NRC Headquarters in Rockville, Maryland from July 21 through July 24, 1997, in order to refine plans for a multi-dimensional characterization of the risk associated with nuclear byproduct material and to develop a statement of

work for contractor evaluation of that risk.

DATES AND TIMES: The working group will meet in closed session on July 21 and 22, 1997, from 7:30 a.m. to 4:00 p.m., to develop the statement of work. The working group will meet in open session on July 23, 1997, from 7:30 a.m. to 4:00 p.m. and on July 24, 1997, from 7:30 a.m. to 12:00 noon, to refine plans for a multi-dimensional characterization of the risk associated with nuclear byproduct material.

ADDRESSES: U.S. Nuclear Regulatory Commission, Two White Flint North, 11545 Rockville Pike, Room T10A17, Rockville, MD, 20852-2738, on Monday, July 21, Tuesday, July 22, and Thursday, July 24, 1997. U.S. Nuclear Regulatory Commission, One White Flint North, 11555 Rockville Pike, Room O1F5, Rockville, MD, 20852-2738, on Wednesday, July 23, 1997.

FOR FURTHER INFORMATION CONTACT: Dennis I. Serig, U.S. Nuclear Regulatory Commission, Office of Nuclear Material Safety and Safeguards, MS T8F5, Washington, DC 20555, telephone (301) 415-7901, e-mail dis@nrc.gov.

SUPPLEMENTARY INFORMATION: As part of the Commission's Strategic Assessment and Rebaselining efforts, an April 15, 1997, SRM related to NRC's Direction Setting Issue 12, Risk-Informed, Performance-Based Regulation, included, in part, Commission direction that the staff reexamine the applicability of risk-informed, performance-based or risk-informed, less prescriptive approaches to regulation of nuclear material. In response to that direction, the Nuclear Byproduct Material Risk Review Group will identify and document a technical basis for a risk-informed approach to the regulation of nuclear byproduct material, and will develop plans for a graded approach to nuclear byproduct material regulation based on risk information. The working group's activities encompass areas addressed in Title 10 of the Code of Federal Regulations Parts 30-36 and 39. The group's approach will involve systematic evaluations of the "nuclear byproduct material systems." Nuclear byproduct material systems will be defined as broadly as is necessary to identify the real-world risks associated with them (encompassing part 39, Licenses and radiation safety requirements for well logging).

The group's approach will involve systematic evaluations of the "nuclear byproduct material systems". Nuclear byproduct material systems will be defined as broadly as is necessary to identify the real-world risks associated with them (ranging from normal use of

the material, to public perceptions during and after an event). The approach will include consideration of the activities of Agreement States' regulatory programs where appropriate, and the group will consider the effects of its recommendations on Agreement States.

Committee Organization and Operations

Dennis Serig, NRC, Office of Nuclear Material Safety and Safeguards, has been selected as chairman. Other members are Elizabeth Ullrich, NRC, Region I; John Lubinski, NRC, Office of Nuclear Material Safety and Safeguards; John Randall, NRC, Office of Nuclear Regulatory Research; and Nancy Daugherty, State of Colorado.

Committee Meetings

The working group will meet at approximately monthly intervals in the Washington, DC, area, or at other locations agreed upon by the working group members. Meetings will be announced in advance through the NRC Public Meeting Notice System and, with some exceptions, will be open for public observation. Persons attending working group meetings will be welcome to provide comments to the working group for its consideration, either in written form or orally, at times specified by the working group chair.

Dated at Rockville, Maryland this 14th day of July, 1997.

For the Nuclear Regulatory Commission.

Frederick C. Combs,

Deputy Director, Division of Industrial and Medical Nuclear Safety, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 97-18994 Filed 7-17-97; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-244]

Rochester Gas and Electric Corporation; R.E. Ginna Nuclear Power Plant; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of its regulations for Facility Operating License No. DRP-18 issued to Rochester Gas and Electric Corporation (the licensee), for operation of the R.E. Ginna Nuclear Power Plant located in Wayne County, New York.

Environmental Assessment

Identification of Proposed Action

The proposed action would exempt Rochester Gas and Electric Corporation from the requirements of 10 CFR 70.24, which requires a monitoring system that will energize clear audible alarms if accidental criticality occurs in each area in which special nuclear material is handled, used, or stored. The proposed action would also exempt the licensee from the requirements to maintain emergency procedures for each area in which this licensed special nuclear material is handled, used, or stored to ensure that all personnel withdraw to an area of safety upon the sounding of the alarm, to familiarize personnel with the evacuation plan, and to designate responsible individuals for determining the cause of the alarm, and to place radiation survey instruments in accessible locations for use in such an emergency.

The proposed action is in accordance with the licensee's application for exemption dated June 5, 1997.

The Need for the Proposed Action

The purpose of 10 CFR 70.24 is to ensure that if a criticality were to occur during the handling of special nuclear material, personnel would be alerted to that fact and would take appropriate action. At a commercial nuclear power plant the inadvertent criticality with which 10 CFR 70.24 is concerned could occur during fuel handling operations. The special nuclear material that could be assembled into a critical mass at a commercial nuclear power plant is in the form of nuclear fuel; the quantity of other forms of special nuclear material that is stored on site is small enough to preclude achieving a critical mass. Because the fuel is not enriched beyond 5.0 weight percent Uranium-235 and because commercial nuclear plant licensees have procedures and design features that prevent inadvertent criticality, the staff has determined that an inadvertent criticality would not likely occur due to the handling of special nuclear material at a commercial power reactor. The requirements of 10 CFR 70.24, therefore, are not necessary to ensure the safety of personnel during the handling of special nuclear materials at commercial power reactors.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that there is no significant environmental impact if the exemption is granted. Inadvertent or accidental criticality will be precluded through

compliance with the R.E. Ginna Nuclear Plant Technical Specifications, the design of the fuel storage racks providing geometric spacing of fuel assemblies in their storage locations, and administrative controls imposed on fuel handling procedures. Technical Specifications requirements specify reactivity limits for the fuel storage racks and minimum spacing between the fuel assemblies in the storage racks.

Appendix A of 10 CFR Part 50, "General Design Criteria for Nuclear Power Plants," Criterion 62, requires the criticality in the fuel storage and handling system shall be prevented by physical systems or processes, preferably by use of geometrically-safe configurations. This is met at Ginna, as identified in the Technical Specifications and the Updated Final Safety Analysis Report (UFSAR). Ginna Technical Specifications Section 4.3, Fuel Storage, states that the spent fuel storage racks are designed with $K_{\text{eff}} \leq 0.95$ if fully flooded with unborated water; and new fuel racks are designed with $K_{\text{eff}} \leq 0.95$ if fully flooded with unborated water; and $K_{\text{eff}} \leq 0.98$ if moderated by aqueous foam. UFSAR Section 9.1.1, New Fuel Storage, states that the spacing of new fuel assemblies ensures a K_{eff} less than 0.95 for the accidental full water density flooding scenario and less than 0.98 for the accidental low water density (optimum moderation) flooding scenario.

The proposed exemption would not result in any significant radiological impacts. The proposed exemption would not affect radiological plant effluent nor cause any significant occupational exposures since the Technical Specifications, design controls (including geometric spacing of fuel assembly storage spaces) and administrative controls preclude inadvertent criticality. The amount of radioactive waste would not be changed by the proposed exemption.

The proposed exemption does not result in any significant nonradiological environmental impacts. The proposed exemption involves features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission has concluded that there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental