

For the Nuclear Regulatory Commission.
 Louis L. Wheeler,
Senior Project Manager, Project Directorate II-2, Division of Reactor Projects - I/II, Office of Nuclear Reactor Regulation.
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[Docket No. 50-305]

Wisconsin Public Service Company, Wisconsin Power and Light Company and Madison Gas and Electric Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR-43 issued to Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensee), for operation of the Kewaunee Nuclear Power Plant, located in Kewaunee County, Wisconsin.

The proposed amendment would change Technical Specification (TS) requirements related to the low temperature overpressure protection (LTOP) system. Specifically, the LTOP curve would be modified to define 10 CFR Part 50, Appendix G pressure temperature limitations for LTOP evaluation through the end of operating cycle (EOC) 33. In addition, the LTOP enabling temperature and the temperature required for starting a reactor coolant pump would be changed consistent with the design basis for the LTOP system. Finally, the TS bases would be changed consistent with the changes described above.

In a letter dated September 27, 1996, the licensee requested that this amendment application be treated exigently. The current LTOP curve is applicable through EOC 21 or 18.40 effective full-power years (EFPY). The startup for cycle 22 is scheduled for October 22, 1996. Due to time constraints, sufficient time is not available to permit the customary public notice in advance of this action. This proposed amendment supersedes a previously submitted proposed amendment on this subject dated April 30, 1996, which was published in the Federal Register on May 22, 1996 (61 FR 25714). The new submittal was necessary in order to address NRC concerns with the original submittal.

Before issuance of the proposed license amendment, the Commission

will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The proposed change was reviewed in accordance with the provisions of 10 CFR 50.92 to show no significant hazards exist. The proposed change will not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The LTOP setpoint, revised enabling temperature, and revised P/T [pressure/temperature] limits reflected in proposed Figure TS 3.1-4 ensure that the Appendix G pressure/temperature limits are not exceeded, and therefore, help ensure that RCS [reactor coolant system] integrity is maintained. The changes do not modify the reactor coolant system pressure boundary, nor make any physical changes to the facility design, material, construction standards, or setpoints. The LTOP valve setpoint remains at ≤ 500 psig. The LTOP enabling temperature based on Figure TS 3.1-4 is 355 °F and is consistent with BTP RSB 5-2 guidance of $RT_{NDT} + 90$ °F. The revised enabling temperature is greater than the 338 °F value in the current TS. A higher enabling temperature ensures that the LTOP system is available for the prevention of non-ductile failure over a larger operating window. The probability of a LTOP event occurring is independent of the pressure-temperature limits for the RCS pressure boundary and enabling temperature. Therefore, the probability of a LTOP event is not increased.

The calculation of pressure temperature limits in accordance with approved regulatory methods provides assurance that reactor pressure vessel fracture toughness requirements are met and the integrity of the RCS pressure boundary is maintained. Similar methodology was used in calculations to support approved amendment 120 to the Kewaunee Technical Specifications dated April 26, 1995. The material property bases, including chemistry factor and initial reference temperature for the unirradiated material (RT_{NDT}), and margin terms, used for this PA are more conservative than that used in the current TS.

The PT limits reflected in proposed Figure TS 3.1-4 are based on the following criteria:

(a) An initial RT_{NDT} value of -56 °F. Drop weight testing of Kewaunee surveillance material was performed by the Westinghouse Electric Corporation and documented in WCAP 14042, Revision 1, dated January 1995 with a resultant initial RT_{NDT} of -50 °F. Testing of sister plant surveillance material resulted in an initial RT_{NDT} of -30 °F. The mean value for all Linde 1092 weld heats in -50.7 °F. Therefore, use of the generic value of -56 °F (for welds made with Linde 1092 flux) with a larger margin term was deemed more conservative and acceptable for this evaluation.

(b) Paragraph (c)(2)(ii)(A) of 10 CFR 50.61. Paragraph (c)(2)(ii)(A) of 10 CFR 50.61 requires that licensees determine a material-specific value of chemistry factor when the surveillance data is deemed credible according to the criteria of paragraph (c)(2)(I) of 10 CFR 50.61. Reference 3 documents WPSC's evaluation which concludes that the KNPP surveillance capsule data satisfy the credibility criteria. The calculated material-specific chemistry factor value is 190.6 °F (based on KNPP surveillance capsule data from capsules V, R, P, and S). Adjustment of this chemistry factor has been accomplished by multiplying by 1.18, the ratio of the best estimate chemistry factor for heat IP3571 to the chemistry factor for the Kewaunee surveillance weld. This results in a chemistry factor value of 224.9 °F.

(c) Neutron fluence (E greater than 1 MeV) projections through [the] end of operating cycle 33. The use of predicted fluence values through the end of operating cycle 33 is appropriately considered within the calculations in accordance with standard industry methodology previously docketed under WCAP 13227 and WCAP 14279. The neutron exposure projections utilized for calculation of the reference temperature were multiplied by a factor of 1.11 to adjust for biases observed between cycle specific calculations and the results of neutron dosimetry for the four surveillance capsules removed from the KNPP reactor. The factor of 1.11 was derived by taking the average of the measured to calculation (M/C) flux ratios obtained from the dosimetry results of capsules V, R, P, and S removed from the KNPP reactor vessel. The resulting effect of using predicted fluence values through the end of cycle 33 instead of cycle 21 is to require the [plant to evaluate LTOP transients to more limiting requirements].

Additional conservatism from a more conservative material property basis and higher projected fluence values is readily illustrated by the increase in magnitude of $EOC_{NDT1/4T}$ from 212.94 °F (derived from the material property basis used in the current TS) to 264.46 °F used for this PA. The proposed PT limits are shifted to a lower pressure and higher temperature, which is more conservative.

The changes do not adversely affect the integrity of the RCS such that its function in the control of radiological consequences is affected. In addition, the changes do not affect any fission barrier. The changes do not degrade or prevent the response of the LTOP relief valve or other safety-related systems to

previously evaluated accidents. In addition, the changes do not alter any assumption previously made in the radiological consequences evaluations nor affect the mitigation of the radiological consequences of an accident previously evaluated. Therefore, the consequences of an accident previously evaluated will not be increased.

Thus, operation of KNPP in accordance with the PA does not involve a significant increase in the probability or consequences of any accident previously evaluated.

2. Create the possibility of a new or different type of accident from an accident previously evaluated.

The enabling temperature and Appendix G pressure temperature limitations were prepared using methods derived from the ASME Boiler and Pressure Vessel Code and the criteria set forth in NRC Regulatory Standard Review Plan 5.3.2. The changes do not cause the initiation of any accident nor create any new credible limiting failure for safety-related systems and components. The changes do not result in any event previously deemed incredible being made credible. As such, it does not create the possibility of an accident different than previously evaluated.

The changes do not have any adverse effect on the ability of the safety-related systems to perform their intended safety functions. Since the enabling temperature is higher, the LTOP system is available for prevention of non-ductile failure over a wide operating window. The new LTOP operating window (i.e., less than or equal to 355 °F) is within the existing band for the residual heat removal system; operating procedures allow the LTOP system to be placed into service at less than 400 °F. The proposed changes do not make physical changes to the plant or create new failure modes. Therefore, it will not create the possibility of a malfunction of equipment important to safety different than previously evaluated. Thus, the PA does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The use of Paragraph (c)(2)(ii)(A) of 10 CFR 50.61, chemistry factor ratio of 1.18, initial reference temperature of -56 °F, and fluence values through EOC [end of cycle] 33 does not modify the reactor coolant system pressure boundary, nor make any physical changes to the LTOP setpoint or system design. Proposed Figure TS 3.1-4 was prepared in accordance with regulatory requirements and requires evaluation of LTOP events to the more conservative material property basis and more limiting requirements of neutron exposure projections of 33.41 EFPY instead of 18.40 EFPY.

Therefore, the PA does not create the possibility of a new or different type of accident from any accident previously evaluated.

3. Involve a significant reduction in the margin of safety.

The Appendix G pressure temperature limitations were prepared using methods derived from the ASME Boiler and Pressure Vessel Code and the criteria set forth in NRC Regulatory Standard Review Plan 5.3.2. These documents along with the calculational limitations specified in 10 CFR 50.61 are an acceptable method for

implementing the requirements of 10 CFR 50 Appendices G and H. Inherent conservatism in the P/T limits resulting from these documents include:

a. An assumed defect in the reactor vessel wall with a depth equal to $\frac{1}{4}$ of the thickness of the vessel wall ($\frac{1}{4}T$) and a length equal to $1\frac{1}{2}$ times the thickness of the vessel wall.

b. Assumed reference flaw oriented in both longitudinal and circumferential directions and limiting material property. At KNPP, the only weld in the core region is oriented in the circumferential direction.

c. A factor of safety of 2 is applied to the membrane stress intensity factor.

d. The limiting toughness is based upon a reference value (K_{IR}) which is a lower bound on the dynamic crack initiation or arrest toughness.

e. A 2-sigma margin term is applied in determining the adjusted reference temperature (ART) that is used to calculate the limiting toughness.

Similar methodology was used in calculations to support approved amendment 120 dated April 26, 1995. Beyond the conservatism described above, WPSC [Wisconsin Public Service Corporation] has incorporated the following additional margin in preparing this PA:

a. The reactor coolant pump starting restrictions of TS 3.1.a.1.c reflect the more limiting LTOP enabling temperature of 355 °F consistent with the design basis for the LTOP system.

b. The LTOP enabling temperature based on Figure TS 3.1-4 is 355 °F and is more conservative than the 338oF value in the current TS.

c. The calculated material-specific chemistry factor value of 190.6oF (based upon KNPP surveillance capsule data from capsules V, R, P, and S) has been multiplied by 1.18 yielding an adjusted chemistry factor value of 224.9oF to account for chemical composition differences between the best estimate value for weld heat IP3571 and the Kewaunee surveillance weld material. d. The neutron exposure projections were multiplied by a factor of 1.11 to adjust for biases observed between cycle specific calculations and the results of neutron dosimetry for the four surveillance capsules removed from the KNPP reactor. The factor of 1.11 was derived by taking the average of the measured to calculation (M/C) flux ratios obtained from the dosimetry results of capsules V, R, P, and S removed from the KNPP reactor vessel. Additional conservatisms beyond that described above but not used in development of the proposed TS and Figure include: (a) A 2 inch diameter spring loaded safety valve set at 480 psig located in the LTOP system. At 500 psig, the LTOP relief valve setpoint, the relieving capacity of this smaller valve is 230 gpm. (b) The actual LTOP relief valve capacity is at least 10% greater than the capacity used in the design and setpoint analyses. This is in accordance with the requirements of Section III NC-7000. (c) Assumptions in the overpressure transient analyses are conservative relative to the actual Kewaunee reactor coolant system (RCS) and operating practices:

1. The RCS was assumed to be rigid with respect to metal expansion.

2. No credit was taken for the shrinkage effect caused by low temperature safety injection water added to higher temperature reactor coolant.

3. No credit was taken for the reduction in reactor coolant bulk modulus at RCS temperatures above 100°F (constant bulk modulus at all RCS temperatures).

4. The entire volume of water of the steam generator secondary was assumed available for heat transfer to the primary. In reality, the liquid immediately adjacent and above the tube bundle would be the primary source of energy in the transient.

5. The overall steam generator heat transfer coefficient, U, was assumed to be the free convective heat transfer coefficient of the secondary, h_{sec} . The forced convective heat transfer coefficient of the primary, h_{pri} and the tube metal resistance have been ignored thus resulting in a conservative (high) coefficient.

6. The reactor coolant pump start time assumed in the heat input analysis was 9-10 seconds; whereas, the Kewaunee pump startup time is 25-30 seconds.

An alternative methodology to the safety margins required by Appendix G to 10 CFR Part 50 has been developed by the ASME Working Group on Operating Plant Criteria. This methodology is contained in ASME Code Case N-514. The Code Case N-514 provides criteria to determine pressure limits during LTOP events that avoid certain unnecessary operational restrictions, provide adequate margins against failure of the reactor pressure vessel, and reduce the potential for unnecessary activation of the relief valve used for LTOP. Specifically, the ASME Code Case N-514 allows determination of the setpoint for LTOP events such that the maximum pressure in the vessel would not exceed 110% of the P/T limits of the existing ASME Appendix G; and redefines the enabling temperature at a coolant temperature less than 200°F or a reactor vessel metal temperature less than $RT_{NDT} + 50°F$, whichever is greater. Code Case N-514, "Low Temperature Overpressure Protection," has been approved by the ASME Code Committee but not yet approved for use in Regulatory Guide 1.147. The content of this code case has been incorporated into Appendix G of Section XI of the ASME Code and published in the 1993 Addenda to Section XI. It is expected that next revision of 10 CFR 50.55a will endorse the 1993 Addenda and Appendix G of Section XI. As stated above, this PA utilizes Appendix G limits and an enabling temperature corresponding to a reactor vessel metal temperature less than $RT_{NDT} + 90°F$, which is more conservative than the alternative methodology contained in Code Case N-514.

The revised calculations meet the NRC acceptance criteria for the LTOP setpoint and system design as described in NRC Safety Evaluation Report (SER) dated September 6, 1985 which concluded that "the spectrum of postulated pressure transients would be mitigated * * * such that the temperature pressure limits of Appendix G to 10 CFR 50 are maintained."

Use of the methodology set forth in the ASME Boiler and Pressure Vessel Code, NRC

Regulatory Standard Review Plan 5.3.2, 10 CFR 50.61, and 10 CFR 50 Appendices G and H with the above additional margins ensures that proper limits and safety factors are maintained. Thus, the PA does not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 15 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 15-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in preventing startup of the facility, the Commission may issue the license amendment before the expiration of the 15-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the Federal Register a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By November 6, 1996, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and

any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the University of Wisconsin, Cofrin Library, 2420 Nicolet Drive, Green Bay, Wisconsin 54311-7001. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of

the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri

1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Gail H. Marcus: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Bradley D. Jackson, Esq., Foley and Lardner, P. O. Box 1497, Madison, Wisconsin 53701-1497, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated September 27, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the University of Wisconsin, Cofrin Library, 2420 Nicolet Drive, Green Bay, Wisconsin 54311-7001.

Dated at Rockville, Maryland, this 2nd day of October 1996.

For The Nuclear Regulatory Commission,
Richard J. Laufer,
*Project Manager, Project Directorate III-3,
Division of Reactor Projects—III/IV, Office of
Nuclear Reactor Regulation.*

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Strategic Assessment and Rebaselining Stakeholders Public Meetings

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of meeting.

SUMMARY: The Nuclear Regulatory Commission is requesting public comment on the second phase of a critical evaluation known as strategic assessment and rebaselining initiative. The NRC is utilizing various media and conducting three public conferences in order to reach as many stakeholders as possible. The objectives of these public meetings are to give the public an opportunity to meet with agency representatives and comment on 16

issue papers the Commission has under active consideration.

This effort was initiated in September 1995, and is being completed in four phases with the goal of finalizing a strategic plan in early CY 1997. The development and implementation of this strategic plan will meet the requirements of the Government Performance and Results Act (GPRA) of 1993.

The effort is presently in the latter portion of the second phase where the Commission is considering a variety of options for addressing key strategic issues facing the NRC as it prepares to move into the 21st century. The NRC will be seeking the views and comments of its stakeholders—Federal entities (Administration/OMB, Congress, and other agencies), NRC employees and their representatives, Agreement States, non-Agreement States, compliers (e.g., licensees, employees of licensees, industry groups), public interest groups, and the general public—as part of the decision-making process. The stakeholder involvement effort began in mid-September and concludes on November 15, 1996. The Commission will consider stakeholder comments before making final decisions on the key strategic issues.

During the week of September 16, 1996, the issue papers and other documents dealing with the strategic assessment were made available to the public. Copies of these documents as well as registration and general information on the public meetings can be obtained electronically from the NRC's Home Page on the World Wide Web (Internet address <http://www.nrc.gov>) and FedWorld at 1-800-303-9672. Paper copies are available by calling NRC's Public Document Room at 1-800-397-4209.

To help understand their viewpoints, stakeholders are asked to focus on the following in responding to the NRC:

1. What, if any, important considerations may have been omitted from the issue papers?
2. How accurate are the NRC's assumptions and projections for internal and external factors discussed in the issue papers?
3. Do the Commission's preliminary views associated with each issue paper respond to the current environment and challenges?
4. Additionally, the Commission is seeking comments on specific questions identified in the "Preliminary Commission View" section of each issue paper.

In Phase I, a steering committee comprised of senior agency managers, working with an outside consultant,

reviewed the NRC's activities in order to understand where the NRC is today, and what needs to be considered in providing options for responding to change. Some of the key objectives identified by the steering committee were: establish a strategic framework under which the NRC will continue to meet its primary responsibility of protecting public health and safety and the environment; provide a sound and well-rounded foundation for the NRC's direction and decision-making for the rest of this decade and into the next century; ensure that the Commission, its staff, Congress, other Government agencies, and the public have a common understanding of what the NRC's strategic goals are; and establish agency performance measures to determine the extent to which strategic or tactical objectives are being achieved.

The NRC will hold three public meetings to discuss the issue papers and to obtain comments from stakeholders. The conference dates and locations are:

DATES: October 24-25, Washington, DC—Washington Hilton; October 31—November 1, Colorado Springs, CO—Sheraton Hotel; November 7-8, Chicago, IL—the Ramada O'Hare.

ADDRESSES: The Washington Hilton and Towers; 1919 Connecticut Avenue NW., Washington, DC 20009 (Tel: 202-483-3000; Fax: 202-265-8221); The Sheraton Colorado Springs Hotel; 2886 South Circle Drive; Colorado Springs, CO 80906 (Tel: 719-576-5900; Fax: 719-576-7695); The Ramada Hotel—O'Hare; 6600 N. Mannheim Road; Rosemont, IL 60018 (Tel: 847-827-5131; Fax: 847-827-5659).

Registration Information

Additional information on the agenda, times, and locations of the public meetings is available via Internet as indicated above. Sleeping rooms have been reserved at a special conference rate at each of the hotels. Those planning to attend the meeting(s) should make their own hotel reservations by telephone using a major credit card, and identifying themselves as an attendee of the NRC Public Meeting. There will be no charge for attending these public meetings, and registration will be held onsite.

Miscellaneous questions about registration should be directed to: Anna May Haycraft, U.S. Nuclear Regulatory Commission; Internet: AMH@NRC.gov or Phone: 301-415-3075.

Dated in Rockville, Maryland this 2nd day of October 1996.