in the NRC review of misadministrations.

Submit, by December 6, 1996, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street NW, (lower level), Washington, DC. Members of the public who are in the Washington, DC, area can access this document via modem on the Public Document Room Bulletin Board (NRC's Advanced Copy Document Library), NRC subsystem at FedWorld, 703–321–3339. Members of the public who are located outside of the Washington, DC, area can dial FedWorld, 1–800–303–9672, or use the FedWorld Internet address: fedworld.gov (Telnet). The document will be available on the bulletin board for 30 days after the signature date of this notice. If assistance is needed in accessing the document, please contact the FedWorld help desk at 703–487–4608.

Additional assistance in locating the document is available from the NRC Public Document Room, nationally at 1–800–397–4209, or within the Washington, DC, area at 202–634–3273.

Comments and questions about the information collection requirements may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T–6 F33, Washington, DC, 20555–0001, or by telephone at (301) 415–7233, or by Internet electronic mail at BJS1@NRC.GOV.

Dated at Rockville, Maryland, this 18th day of October, 1996.

For the Nuclear Regulatory Commission.

Gerald F. Cranford,

Designated Senior Official for Information Resources Management.

[FR Doc. 96–25627 Filed 10–04–96; 8:45 am]

BILLING CODE 7590–01–P

[Docket Nos. 50–424 and 50–425]

Georgia Power Company, et al.; Notice of Issuance of Amendments to Facility Operating Licenses

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 96 to Facility Operating License No. NPF–68 and Amendment No. 74 to Facility Operating License No. NPF–81 issued to Georgia Power Company, et al. (the licensee), which revised the Technical Specifications and associated Bases for operation of the Vogtle Electric Generating Plant, Units 1 and 2, located in Burke County, Georgia.

The amendments are effective as of the date of issuance and shall be implemented within 150 days from the date of issuance. Implementation shall include the relocation of Technical Specification requirements to the appropriate licensee-controlled document as identified in the licensee's application dated May 1, 1995, as supplemented by letters dated August 3 and 9, September 22, November 20, and December 21, 1995, January 26 and 30, February 19 and 29, March 5 and 12, May 6, June 17, August 23, and September 13, 1996, and reviewed in the staff's Safety Evaluation dated

The amendments replaced, in its entirety, the current Technical Specifications and associated Bases with a set based on NUREG–1431, ''Standard Technical Specifications, Westinghouse Plants,'' Revision 1, dated April 1995.

There are three specific items in the licensee's application that are still being reviewed by the staff. Two of these items are the allowed outage time (AOT) for the emergency diesel generators (EDGs) and the AOT for the containment spray system. In accordance with supplements to the initial application, these two items are being addressed in the license amendments by retaining the provisions of the licensee's current licensing basis. New licensing actions are being initiated as a means for continuing the staff evaluation of the AOT proposals. Appropriate license amendments will be issued when those reviews are completed.

The third item concerns the staff evaluation of the licensing basis for containment isolation valves in closed systems. The licensee's current licensing basis is being retained pending the resolution of an unresolved inspection item. Any changes needed to Technical Specification 3.6.3 as a result of that review will be addressed in a future licensing action. The licensee will be kept informed of the status of that review in separate correspondence.

All other issues in the licensee's application for Technical Specification conversion are resolved in the license amendments.

The application for the amendments, dated May 1, 1995, as supplemented by letters dated August 3 and 9, September 22, November 20, and December 21, 1995, January 26 and 30, February 19 and 29, March 5 and 12, May 6, June 17, August 23, and September 13, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Opportunity for a Hearing in connection with this action was published in the Federal Register on September 7, 1995 (60 FR 46633) and on January 10, 1996 (61 FR 734). No request for a hearing or petition for leave to intervene was filed following these notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality of the human environment (61 FR 8308).

For further details with respect to the action see (1) the application for amendments dated May 1, 1995, as supplemented by letters dated August 3 and 9, September 22, November 20, and December 21, 1995, January 26 and 30, February 19 and 29, March 5 and 12, May 6, June 17, August 23, and September 13, 1996, (2) Amendment No. 94 to License No. NPF–68 and Amendment No. 72 to License No. NPF–81, (3) the Commission's related Safety Evaluation dated September 25, 1996, and (4) the Commission's Environmental Assessment dated February 27, 1996.

All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Burke County Library, 412 Fourth Street, Waynesboro, Georgia.

Dated at Rockville, Maryland, this 25th day of September 1996.

For the Nuclear Regulatory Commission.

Louis L. Wheeler,

Senior Project Manager, Project Directorate II–2, Division of Reactor Projects - I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 96–25626 Filed 10–4–96; 8:45 am]

BILLING CODE 7590–01–P

[Docket No. 50–305]

Wisconsin Public Service Company, Wisconsin Power and Light Company and Madison Gas and Electric Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR–43 issued to Wisconsin Public Service Corporation, Wisconsin Power and Light Company, and Madison Gas and Electric Company (the licensee), for operation of the Kewaunee Nuclear Power Plant, located in Kewaunee County, Wisconsin.

The proposed amendment would change Technical Specification (TS) requirements related to the low temperature overpressure protection (LTOP) system. Specifically, the LTOP curve would be modified to define 10 CFR Part 50, Appendix G pressure temperature limitations for LTOP evaluation through the end of operating cycle (EOC) 33. In addition, the LTOP enabling temperature and the temperature required for starting a reactor coolant pump would be changed consistent with the design basis for the LTOP system. Finally, the TS bases would be changed consistent with the changes described above.

In a letter dated September 27, 1996, the licensee requested that this amendment application be treated exigently. The current LTOP curve is applicable through EOC 21 or 18.40 effective full-power years (EFPY). The startup for cycle 22 is scheduled for October 22, 1996. Due to time constraints, sufficient time is not available to permit the customary public notice in advance of this action. This proposed amendment supersedes a previously submitted proposed amendment on this subject dated April 30, 1996, which was published in the Federal Register on May 22, 1996 (61 FR 25714). The new submittal was necessary in order to address NRC concerns with the original submittal.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The proposed change was reviewed in accordance with the provisions of 10 CFR 50.92 to show no significant hazards exist. The proposed change will not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The LTOP setpoint, revised enabling temperature, and revised P/T [pressure/temperature] limits reflected in proposed Figure TS 3.1–4 ensure that the Appendix G pressure/temperature limits are not exceeded, and therefore, help ensure that RCS [reactor coolant system] integrity is maintained. The changes do not modify the reactor coolant system pressure boundary, nor make any physical changes to the facility design, material, construction standards, or setpoints. The LTOP valve setpoint remains at ≤ 500 psig. The LTOP enabling temperature based on Figure TS 3.1–4 is 355 °F and is consistent with BTP RSB 5–2 guidance of RT_{NDT} + 90 °F. The revised enabling temperature is greater than the 338 °F value in the current TS. A higher enabling temperature ensures that the LTOP system is available for the prevention of non-ductile failure over a larger operating window. The probability of a LTOP event occurring is independent of the pressure-temperature limits for the RCS pressure boundary and enabling temperature. Therefore, the probability of a LTOP event is not increased.

The calculation of pressure temperature limits in accordance with approved regulatory methods provides assurance that reactor pressure vessel fracture toughness requirements are met and the integrity of the RCS pressure boundary is maintained. Similar methodology was used in calculations to support approved amendment 120 to the Kewaunee Technical Specifications dated April 26, 1995. The material property bases, including chemistry factor and initial reference temperature for the unirradiated material (RT_{NDT}), and margin terms, used for this PA are more conservative than that used in the current TS.

The PT limits reflected in proposed Figure TS 3.1–4 are based on the following criteria:

(a) An initial RT_{NDT} value of –56 °F. Drop weight testing of Kewaunee surveillance material was performed by the Westinghouse Electric Corporation and documented in WCAP 14042, Revision 1, dated January 1995 with a resultant initial RT_{NDT} of –50 °F. Testing of sister plant surveillance material resulted in an initial RT_{NDT} of –30 °F. The mean value for all Linde 1092 weld heats in –50.7 °F. Therefore, use of the generic value of –56 °F (for welds made with Linde 1092 flux) with a larger margin term was deemed more conservative and acceptable for this evaluation.

(b) Paragraph (c)(2)(ii)(A) of 10 CFR 50.61. Paragraph (c)(2)(ii)(A) of 10 CFR 50.61 requires that licensees determine a material-specific value of chemistry factor when the surveillance data is deemed credible according to the criteria of paragraph (c)(2)(I) of 10 CFR 50.61. Reference 3 documents WPSC's evaluation which concludes that the KNPP surveillance capsule data satisfy the credibility criteria. The calculated material-specific chemistry factor value is 190.6 °F (based on KNPP surveillance capsule data from capsules V, R, P, and S). Adjustment of this chemistry factor has been accomplished by multiplying by 1.18, the ratio of the best estimate chemistry factor for heat IP3571 to the chemistry factor for the Kewaunee surveillance weld. This results in a chemistry factor value of 224.9 °F.

(c) Neutron fluence (E greater than 1 MeV) projections through [the] end of operating cycle 33. The use of predicted fluence values through the end of operating cycle 33 is appropriately considered within the calculations in accordance with standard industry methodology previously docketed under WCAP 13227 and WCAP 14279. The neutron exposure projections utilized for calculation of the reference temperature were multiplied by a factor of 1.11 to adjust for biases observed between cycle specific calculations and the results of neutron dosimetry for the four surveillance capsules removed from the KNPP reactor. The factor of 1.11 was derived by taking the average of the measured to calculation (M/C) flux ratios obtained from the dosimetry results of capsules V, R, P, and S removed from the KNPP reactor vessel. The resulting effect of using predicted fluence values through the end of cycle 33 instead of cycle 21 is to require the [plant to evaluate LTOP transients to more limiting requirements].

Additional conservatism from a more conservative material property basis and higher projected fluence values is readily illustrated by the increase in magnitude of $EOC_{NDT1/4T}$ from 212.94 °F (derived from the material property basis used in the current TS) to 264.46oF used for this PA. The proposed PT limits are shifted to a lower pressure and higher temperature, which is more conservative.

The changes do not adversely affect the integrity of the RCS such that its function in the control of radiological consequences is affected. In addition, the changes do not affect any fission barrier. The changes do not degrade or prevent the response of the LTOP relief valve or other safety-related systems to