

Comments are invited on (a) Whether the proposed collection of information is necessary for the proper performance of the functions of the agency, including whether the information shall have practical utility; (b) the accuracy of the agency's estimate of the burden of the proposed collection of information; (c) ways to enhance the quality, utility, and clarity of the information to be collected; and (d) ways to minimize the burden of the collection of information on respondents, including through the use of automated collection techniques or other forms of information technology.

Proposed Project: Graduate students in science, engineering, and health fields in U.S. colleges and universities, by source and mechanism of support and by demographic characteristics—A mail survey, the Survey of Graduate Students and Postdoctorates in Science and Engineering originated in 1966 and has been conducted annually since 1972. The survey is the academic graduate enrollment component of the NSF statistical program that seeks to "provide a central clearinghouse for the collection, interpretation, and analysis of data on the availability of, and the current and projected need for, scientific and technical resources in the United States, and to provide a source of information for policy formulation by other agencies of the Federal government" as mandated in the National Science Foundation Act of 1950. The proposed project will continue the current survey cycle for three to five years. The annual Fall surveys for 1996 through 2000 will survey the universe of approximately 725 institutions offering accredited graduate programs in science, engineering, or health. The survey has provided continuity of statistics on graduate school enrollment and support for graduate students in all science & engineering (S&E) and health fields, with separate data requested on demographic characteristics (race/ethnicity and gender by full-time and part-time enrollment status). Statistics from the survey are published in NSF's annual publication series *Academic Science and Engineering Graduates*, in NSF publications *Science and Engineering Indicators*, *Women, Minorities, and Persons with Disabilities in Science and Engineering*, and are available electronically on the World Wide Web.

The survey will be mailed primarily to the administrators at the Institutional Research Offices. To minimize burden, the NSF is exploring possibilities for using an automatic survey questionnaire (ASQ) diskette, on which institutions

would receive their previous year's data and a complete program for editing and trend checking. Respondents will be encouraged to participate in these initiatives should they so wish. Traditional paper questionnaires will also be available, with editing and trend checking performed as part of the survey processing.

In Fall 1994, the survey achieved a total response rate of 98.9% for institutions and 96.0% for departments.

Burden estimates are as follows:

	Total No. of institutions	Departments	Burden hours
FY 1992	727	10,981	1.76
FY 1993	725	11,134	1.80
FY 1994	724	11,411	1.97

Send comments to Herman Fleming, Clearance Officer, National Science Foundation, 4201 Wilson Boulevard, Suite 485, Arlington, VA 22230. Written comments should be received by September 17, 1996.

Dated: June 17, 1996.
Herman G. Fleming,
NSF Clearance Officer.
[FR Doc. 96-18560 Filed 7-22-96; 8:45 am]
BILLING CODE 7555-01-M

NATIONAL TRANSPORTATION SAFETY BOARD

Sunshine Act Meeting

TIME AND DATE: 9:30 a.m., Tuesday, July 30, 1996.

PLACE: The Board Room, 5th Floor, 490 L'Enfant Plaza, S.W., Washington, D.C. 20594.

STATUS: Open.

MATTERS TO BE CONSIDERED:

6579A Aviation Accident Report:
Uncontained Engine Failure/Fire,
ValuJet Airlines Flight 597, Douglas DC-9-32, Atlanta, Georgia, June 8, 1995.

NEWS MEDIA CONTACT: Telephone: (202) 382-0660.

FOR MORE INFORMATION CONTACT: Bea Hardesty, (202) 382-6525.

Dated: July 19, 1996.
Bea Hardesty,
Federal Register Liaison Officer.
[FR Doc. 96-18812 Filed 7-19-96; 1:52 pm]
BILLING CODE 7533-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-272 and 50-311]

Public Service Electric and Gas Company Salem Nuclear Generating Station, Units 1 and 2; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. DPR-70 and DPR-75 issued to Public Service Electric and Gas Company (the licensee) for operation of the Salem Nuclear Generating Station, Units 1 and 2, located in Salem County, New Jersey.

The proposed amendment would revise Technical Specification (TS) 3.3.2.1, "Engineered Safety Feature Actuation System Instrumentation" to reflect a revised setpoint for the interlock designated P-12.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

This change to the Technical Specifications does not involve any physical changes to the plant or any procedures changes.

There is no safety consequence to the [safety injection] SI function being enabled at 543 °F. The T_{avg} no-load temperature is at 547 °F with increasing T_{avg} for higher power operation. The allowable value of 545 °F as the upper limit assures the availability of the SI function, therefore, the protective function will perform within its analyzed range. On increasing temperature, P-12 automatically

enables SI in both High Steamline Flow coincident with Low-Low T_{avg} and High Steamline Flow coincident with Low Steamline Pressure. It also provides an arming signal to the Steam Dump System.

On decreasing temperature, P-12 permits manual block of SI in both High Steamline Flow coincident with Low-Low T_{avg} and High Steamline Flow coincident with Low Steamline Pressure. This permits blocking of the SI below the minimum temperature for criticality during a controlled shutdown. With a 2 °F allowable deviation from the nominal setpoint, the setpoint of 543 °F is adequate to enable the operator to block SI.

Hardware design of the [engineered safety feature actuation system] ESFAS provides that actuation of the SI block, enable, and ESFAS protection system operations are all provided by the same bistables. The analyses were performed supporting the design of the ESFAS system.

Revision of the P-12 setpoint to enable manual block of SI from 541 °F to 543 °F does not impact the safety analyses. SI is available at or above the T_{avg} no-load value of 547 °F, which is consistent with the setpoint for Low-Low T_{avg} in TS Table 3.3-4. Retaining the allowable value of 541 °F is also consistent with Table 3.3-4. The proposed revisions do not affect the integrity of the fuel assembly or reactor internals such that their function in the control of radiological consequences is affected. In addition, the proposed revisions do not affect any fission product barrier. The proposed revision does not change, degrade, or prevent the response of safety related mitigation systems to accident scenarios, as described in the [Final Safety Analysis Report] FSAR.

Therefore the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes to the TS setpoints for P-12 do not create failure modes that could adversely impact safety-related equipment or cause the initiation of any accident. The P-12 interlock circuit pertains to accident mitigation systems and not accident initiation. Functions of safety related systems and components, which are related to accident mitigation, have not been altered.

The proposed TS setpoint change does not cause the initiation of any accident or create any new credible failure in the system. The proposed revisions do not result in any malfunction of equipment previously evaluated. The proposed revisions do not result in increased probability of equipment failure scenarios previously deemed improbable.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated, the revisions will not create the possibility of a malfunction of equipment important to safety different than previously evaluated in the FSAR.

3. The proposed change does not involve a significant reduction in a margin of safety.

Several different steamline break analyses are performed to support operation of the Salem units. Analyses are performed to determine the core response to postulated steamline breaks and to calculate mass and energy releases both inside and outside containment.

In the current licensing basis core response steamline break analysis, the High Steamline Flow coincident with Low-Low T_{avg} or Low Steamline Pressure protective functions are not modeled. As such, a change to the SI permissive has no impact on the analysis. Other SI signals generated from a postulated steamline break are credited in the analysis. Interlock P-12 is independent of these credited SI signals. Therefore, this change has no impact on the safety analysis.

The licensing basis steamline break mass and energy release safety analyses, inside and outside containment, for Salem Units 1 and 2 assumes the availability of the High Steamline Flow coincident with Low-Low T_{avg} or Low Steamline Pressure for actuation of SI and steamline isolation. However, no credit is taken for these trip functions. The noted Technical Specification change is resolving a discrepancy between the permissive P-12 setpoint and the Low-Low T_{avg} setpoint. Even though this Low-Low T_{avg} function is available in the steamline break mass and energy release analyses, operation is not credited in the analyses.

There are no new safety analyses or revision[s] to any existing safety analyses as a result of these changes. In addition, the proposed change does not impact any input assumptions or results of any current licensing basis analyses for the design basis events. Therefore, there is no significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should

the Commission take this action, it will publish in the Federal Register a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By August 22, 1996 the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons

why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no

significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to John F. Stolz: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal Register notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Mark J. Wetterhahn, Esquire, Winston and Strawn, 1400 L Street, NW., Washington, DC 20005-3502, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1) (i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated July 12, 1996, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey.

Dated at Rockville, Maryland, this 17th day of July 1996.

For the Nuclear Regulatory Commission.
Donald S. Brinkman,
Senior Project Manager, Project Directorate I-2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 96-18665 Filed 7-22-96; 8:45 am]

BILLING CODE 7590-01-P

Sunshine Act Meeting

AGENCY HOLDING THE MEETING: Nuclear Regulatory Commission.

DATES: Weeks of July 22, 29, August 5, and 12, 1995.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and Closed.

MATTERS TO BE CONSIDERED:

Week of July 22

There are no meetings scheduled for the Week of July 22.

Week of July 29—Tentative

Monday, July 29

10:00 a.m.—Briefing on Uranium Recovery Program (Public Meeting) (Contact: Joe Holonich, 301-415-6643)

Tuesday, July 30

10:00 a.m.—Briefing by Nuclear Waste Technical Review Board (Public Meeting)

11:30 a.m.—Affirmation Session (Public Meeting) (if needed)

2:00 p.m.—Briefing on Status of Staff Actions on Industry Restructuring and Deregulation (Public Meeting) (Contact: Dave Mathews, 301-415-1282)

Wednesday, July 31

2:00 p.m.—Briefing on EEO Program (Public Meeting) (Contact: Ed Tucker, 301-415-7382)

Thursday, August 1

3:00 p.m.—Briefing on Spent Fuel Pool Cooling Issues (Public Meeting) (Contact: George Hubbard, 301-415-2870)

Week of August 5—Tentative

There are no meetings scheduled for the Week of August 5.

Week of August 12—Tentative

There are no meetings scheduled for the Week of August 12.

The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings call (Recording)—(301) 415-1292. Contact person for more information: Bill Hill (301) 415-1661.

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The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/SECY/smj/schedule.htm>.

This notice is distributed by mail to several hundred subscribers; if you no longer wish to receive it, or would like to be added to it,