

Dated: June 7, 1996.  
 Stephen H. Greene,  
*Deputy Administrator.*  
 [FR Doc. 96-14953 Filed 6-12-96; 8:45 am]  
 BILLING CODE 4410-09-M

## National Institute of Corrections

### Advisory Board Meeting

*Time and Date:* 12:30 p.m., Monday, June 24, 1996.

*Place:* Ramada Inn, 1117 Williston Road, Burlington, Vermont.

*Status:* Open.

*Matters To Be Considered:* Update on the reimbursement plan for NIC services, Office of Justice Programs' update on the Violent Offender and Truth In Sentencing Grant Program, update on the District of Columbia Department of Corrections Study, progress report from the task force on prison construction standardization and techniques, update on the NIC Executive Excellence Program, status of the final report on the mental health in jails survey, status report on the positional asphyxia video proposal, and the FY 1997 program plan recommendations.

*Contact Person for More Information:* Larry Solomon, Deputy Director, (202) 307-3106, ext. 155.

Morris L. Thigpen,

*Deputy Director.*

[FR Doc. 96-15009 Filed 6-12-96; 8:45 am]

BILLING CODE 4410-36-M

## NATIONAL SCIENCE FOUNDATION

### Special Emphasis Panel in Chemistry; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92-463, as amended), the National Science Foundation announces the following meeting:

*Name:* Special Emphasis Panel in Chemistry (#1191).

*Date and Time:* July 1-2, 1996, 8:00 am to 5:00 pm.

*Place:* University of Texas, Austin, TX.

*Type of Meeting:* Closed.

*Contact Person:* Karolyn K. Eisenstein, Program Director, Office of Special Projects, Chemistry Division, Room 1055, National Science Foundation, 4201 Wilson Blvd. Arlington, VA 22230, Telephone: (703) 306-1850.

*Purpose of Meeting:* To review the renewal proposal, evaluate the Science and Technology Center, and make a recommendation concerning future funding of the Science and Technology Center.

*Agenda:* To evaluate: (a) the research program; (b) educational and outreach activities; and (3) the knowledge transfer

activities and the management of the STC. To make a recommendation on the future funding of the STC.

*Reason for Closing:* The proposal being reviewed includes information of a proprietary or confidential nature, including technical information, financial data, such as salaries; and personal information concerning individuals associated with the proposals. These matters are within exemptions (4) and (6) of 5 U.S.C. 552b(c), the Government in the Sunshine Act.

Dated: June 10, 1996.

[FR Doc. 96-15012 Filed 6-12-96; 8:45 am]

BILLING CODE 7555-01-M

## NATIONAL TRANSPORTATION SAFETY BOARD

### Public Hearing in Maryland: Railroad Accident

In connection with its investigation of the collision and derailment of a MARC commuter passenger train with AMTRAK Train 29, The Capitol Limited, near Silver Spring, Maryland, on February 16, 1996, the National Transportation Safety Board will convene a public hearing at 9:00 a.m. (local time), on June 26, 1996, at the DoubleTree Hotel, 1750 Rockville Pike (Route 355), Rockville, Maryland 20852. For more information, contact Pat Cariseo, Office of Public Affairs, Washington, D.C. 20594, telephone (202) 382-0660.

Dated: June 7, 1996.

Bea Hardesty,

*Federal Register Liaison Officer.*

[FR Doc. 96-14997 Filed 6-12-96; 8:45 am]

BILLING CODE 7533-01-P

## NUCLEAR REGULATORY COMMISSION

[Docket No. 50-261]

### Carolina Power & Light Company, H.B. Robinson Steam Electric Plant, Unit No. 2; Exemption

I

Carolina Power and Light Company (CP&L or the licensee) is the holder of Facility Operating License No. DPR-23, which authorizes operation of the H.B. Robinson Steam Electric Plant, Unit No. 2 (HBR), at steady-state reactor power level not in excess of 2300 megawatts thermal. The facility consists of one pressurized water reactor located at the licensee's site in Darlington County, South Carolina. The license provides, among other things, that it is subject to all rules, regulations and Orders of the Nuclear Regulatory Commission (the

Commission or NRC) now or hereafter in effect.

II

Section III.J of Appendix R to Part 50 of Title 10 of the Code of Federal Regulations (10 CFR Part 50) requires that emergency lighting units with at least an 8-hour battery power supply be provided in all areas needed for operation of post-fire safe shutdown equipment and in access and egress routes thereto. The NRC may grant exemptions from the requirements of the regulations which, pursuant to 10 CFR 50.12(a), are (1) Authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security; and (2) present special circumstances. Special circumstances exist whenever, according to 10 CFR 50.12(a)(2)(ii), "Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule \* \* \*."

III

By letters dated February 2, 1995, May 15, 1995, and September 29, 1995, Carolina Power & Light Company (the licensee), requested an exemption from certain technical requirements of Section III.J of Appendix R to 10 CFR Part 50 for HBR. Section III.J of Appendix R requires that emergency lighting units with at least an 8-hour battery power supply be provided in all areas needed for operation of post-fire safe shutdown equipment and in access and egress routes thereto. The licensee plans to implement procedure enhancements to its post-accident safe shutdown procedure that would require manual operation of certain valves. The licensee proposed to use the diesel-backed security lighting system for access and egress to, and operation of, auxiliary feedwater (AFW) valves AFW-1 and AFW-104 and instrument air (IA) valve IA-297, stating that the use of the diesel-backed security lighting system will provide an equivalent level of fire safety to that achieved by compliance with Section III.J of Appendix R to 10 CFR Part 50 for access and egress to, and operation of, valves AFW-1 and AFW-104, located in fire zone 33, and valve IA-297, located in fire zone 25.

IV

Valves AFW-1, AFW-104, and IA-297 are located in outdoor areas within the protected area. These areas and the access and egress paths do not have 8-hour fixed battery-operated emergency

lighting as required by Section III.J of Appendix R to 10 CFR Part 50.

Valves AFW-1 and AFW-104 are located in fire zone 33 at the bottom of the condensate storage tank (CST). Failure to manually isolate valves AFW-1 and AFW-104 could result in overfilling the CST with service water after switchover of the AFW cooling source from the CST to the service water system. These AFW valves are located in outdoor areas within the protected area and are provided with lighting from the security lighting system. However, the licensee may need to manually operate these valves during the hours of darkness. Because the security lighting system is also backed by a standby diesel generator, the licensee would like to rely on the security lighting system with its standby diesel generator as an acceptable alternative fire protection configuration equivalent to that achieved by conformance with the requirements of 10 CFR Part 50, Appendix R, Section III.J.

Valve IA-297 is located in an outdoor area within the protected area near the southeast corner of the turbine mezzanine in fire zone 25 next to the steam dump accumulator. The HBR safe shutdown analysis takes credit for the availability of the main steam safety valves. The use of the nitrogen backup to the main steam power-operated relief valves is a contingency evolution for coping with a fire in the charging pump room. The licensee has already committed to provide emergency lighting with at least an 8-hour battery power supply for the main steam isolation and relief valve area, also in the southeast corner of the turbine building mezzanine. The licensee would rely on that planned emergency lighting in the vicinity of IA-197 as well as the security lighting system with its backup diesel generator to ensure the light necessary to take the actions described by the licensee.

The manual actions would be limited to operating valves AFW-1, AFW-104, and IA-297 over their full travel, with no requirement to partially open or close a valve, by relying on instruments, or other means, to determine valve travel.

In the licensee's submittal of September 29, 1995, the licensee confirmed that a walkdown was conducted in the areas for which the exemption was requested. With the normal lighting turned off, the light provided solely by the security lighting system was adequate for access and egress to, and operation of, valves AFW-1, AFW-104, and IA-297. During a telephone conference call on December 1, 1995, the licensee

confirmed that postulated fires requiring the operators to travel to and operate valves AFW-1, AFW-104, and IA-297 would not affect the security lighting system. In addition, the security lighting system is backed by a standby diesel generator that has been very reliable; records indicate only two failures in 250 surveillance starts. However, none of the failures were failures to start but, rather, failures to come up to rated speed within the prescribed time of 10 seconds. Should the diesel fail to start, procedure OP-926, "TSC/ EOF/ PAT Diesel Generator," provides instruction for manually starting the diesel, and such an action would be initiated by a call to the control room operators. Therefore, in the event of a fire that requires manual operation of valves AFW-1, AFW-104, and IA-297, or in the event of a loss of offsite power, there is reasonable assurance that the security lighting system will be available and will provide the light necessary to take the actions described above.

On the basis of this evaluation, the NRC staff has concluded that the use of the diesel-backed security lighting system will provide an equivalent level of fire safety to that achieved by compliance with Section III.J of Appendix R to 10 CFR Part 50 for access and egress to, and operation of, valves AFW-1 and AFW-104, located in fire zone 33, and valve IA-297, located in fire zone 25.

The Commission, thus, has determined that, pursuant to 10 CFR 50.12, the exemption requested by the licensee's letters dated February 2, 1995, May 15, 1995, and September 29, 1995, as discussed above, is authorized by law and will not endanger life or property and is otherwise in the public interest. Furthermore, the Commission has determined, pursuant to 10 CFR 50.12(a), that special circumstances as set forth in 10 CFR 50.12(a)(ii) are present and applicable in that application of the regulation in these particular circumstances is not necessary to achieve the underlying purpose of the rule.

The Commission hereby grants an exemption from the technical requirements of 10 CFR Part 50, Appendix R, Section III.J, for the use of the diesel-backed security lighting system for access and egress to, and operation of, valves AFW-1 and AFW-104 and IA-297.

Pursuant to 10 CFR 51.32, the Commission has determined that granting of this exemption will have no significant effect on the quality of the human environment (61 FR 6044). This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 3rd day of June 1996.

For the Nuclear Regulatory Commission.  
Steven A. Varga,  
*Director, Division of Reactor Projects—I/II,  
Office of Nuclear Reactor Regulation.*  
[FR Doc. 96-15017 Filed 6-12-96; 8:45 am]  
BILLING CODE 7590-01-P

**[Docket No. 50-219]**

**GPU Nuclear Corporation; Notice of Denial of Amendment to Facility Operating License and Opportunity for Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) has denied a request by GPU Nuclear Corporation, (licensee) for an amendment to Facility Operating License No. DPR-16 issued to the licensee for operation of the Oyster Creek Nuclear Generating Station, located in Forked River, New Jersey. Notice of Consideration of Issuance of this amendment was published in the Federal Register on December 7, 1995, (60 FR 62895).

The purpose of the licensee's amendment request was to modify the License Condition 2.C(5) to utilize a visual inspection technique in accordance with the American Society of Mechanical Engineers (ASME) Code, Section XI, and to eliminate the requirements to docket inspection results and the need to obtain NRC restart authorization for each refueling outage.

The NRC staff has concluded that the licensee's request cannot be granted and has advised the licensee that the proposed amendment is denied because the licensee has not provided adequate justification to resolve the staff's concern over the long-term behavior of the core spray sparger system. The licensee was notified of the Commission's denial of the proposed change by a letter dated June 7, 1996.

By July 15, 1996, the licensee may demand a hearing with respect to the denial described above. Any person whose interest may be affected by this proceeding may file a written petition for leave to intervene.

A request for hearing or petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date.

A copy of any petitions should also be sent to the Office of the General